

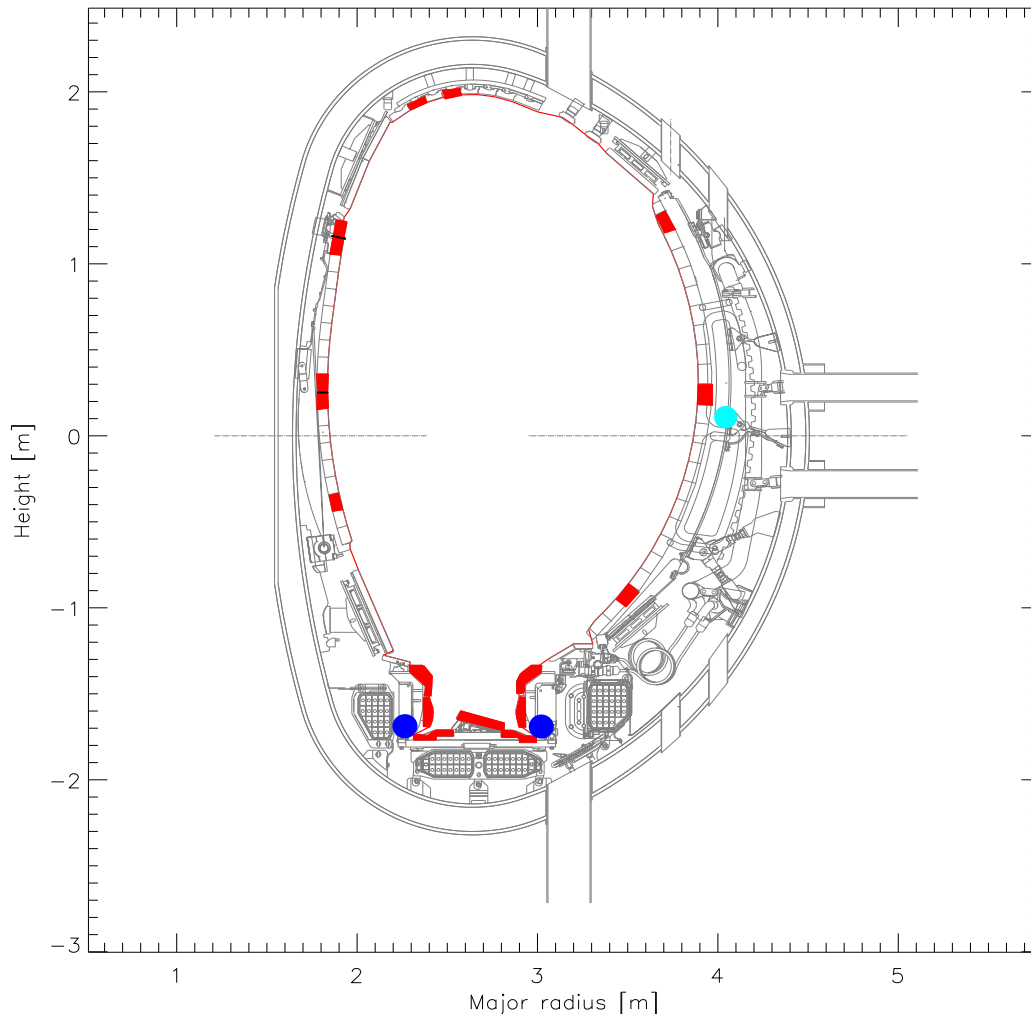


Material Migration Patterns and Overview of First Surface Analysis of the JET ITER-like Wall

***Anna Widdowson, E Alves, C F Ayres, A Baron-Wiechec,
S Brezinsek, J P Coad, K Heinola, J Likonen, G F Matthews,
M Rubel and JET-EFDA contributors***

***14th International Conference on Plasma Facing Materials and Components
14-17 May 2013***

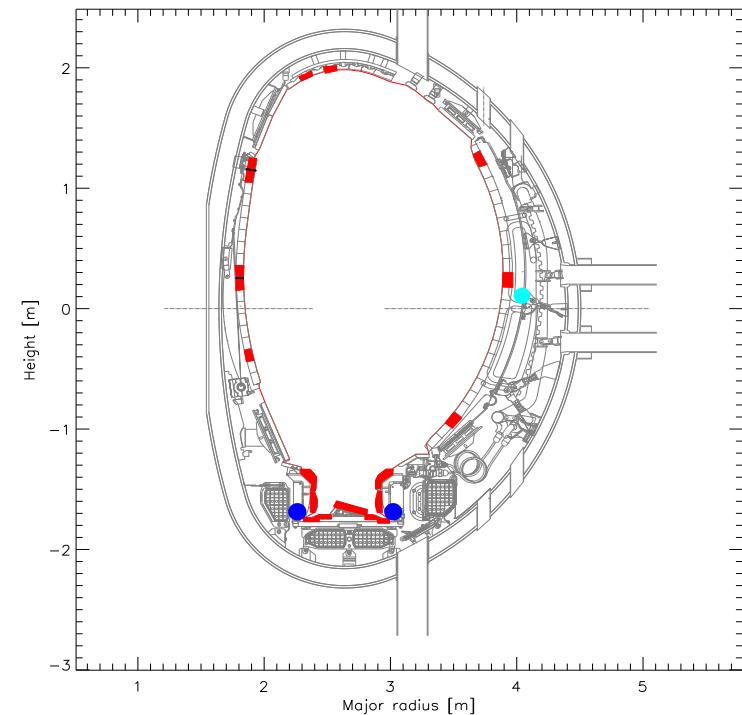
- First JET ITER-Like Wall Campaign 2011-2012
- Intervention
 - Remove and replace tiles from divertor and main chamber
 - Remove and replace erosion/deposition diagnostics
 - Detailed in-vessel photographic survey



- Marker tiles
- Passive diagnostics in remote divertor: Rotating collectors, Test mirrors, louvre clips
- Passive diagnostics outer wall: Rotating collectors, Test mirrors, louvre clips

- Post-mortem analysis of tiles and diagnostics is an on-going analysis programme lasting 2 years
- First results from inner wall limiters and divertor

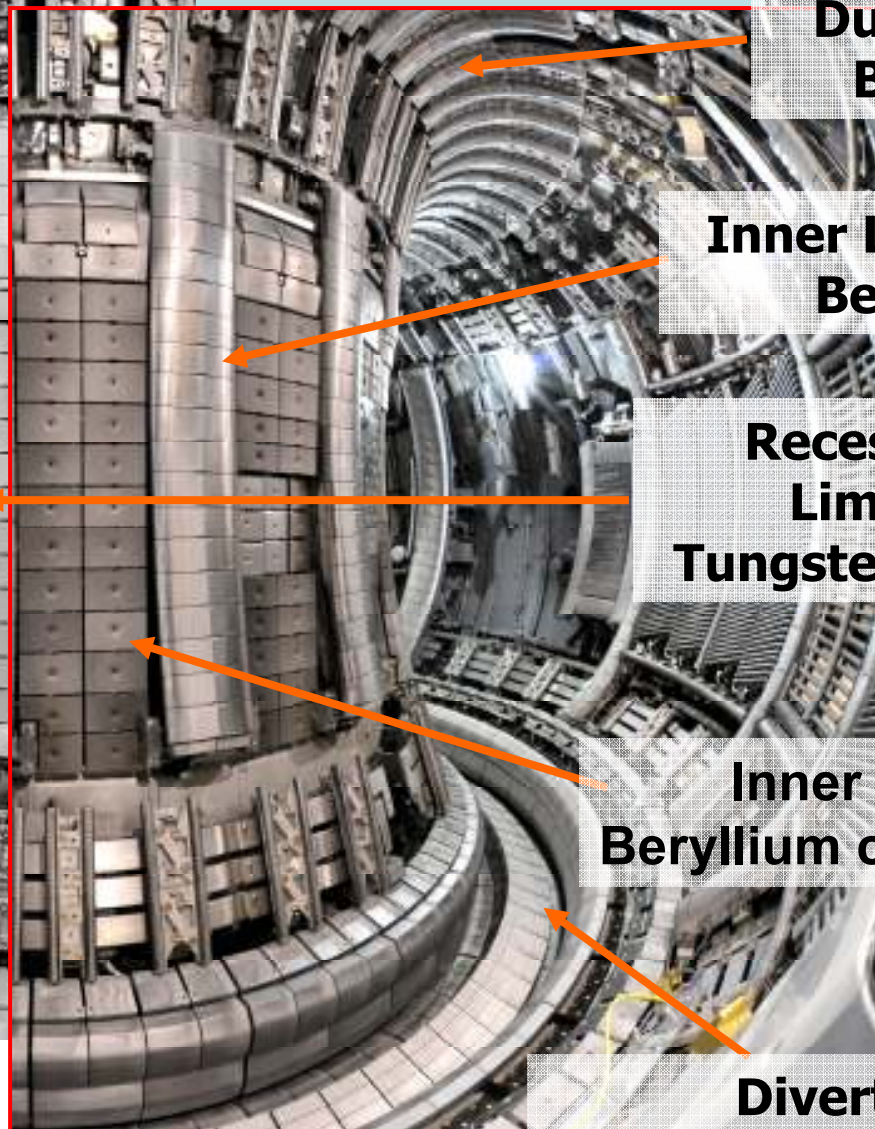
- Changes to tiles during the ILW campaigns
- First erosion and deposition data on tiles removed
 - Inner Wall Guard limiters
 - Dump plates
 - Outer Poloidal Limiters
 - Divertor tiles
 - Diagnostics in remote areas



ILW – Before operations



ILW – After operations



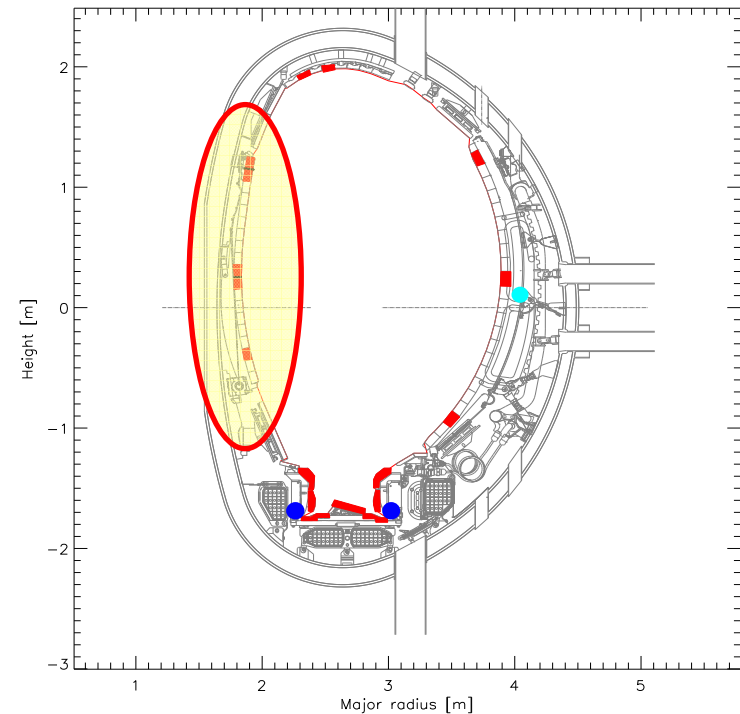
Inner and Outer Divertor tiles
Tungsten coated CFC

Divertor tile
Bulk Tungsten Lamellae

	Total plasma time (h)	X-point (h)	Limiter (h)
2005-2007	35	27	8
2008-2009	45	33	12
2011-2012	19	13	6




- During ILW operations there were dedicated limited plasma to understand the limits of the beryllium wall

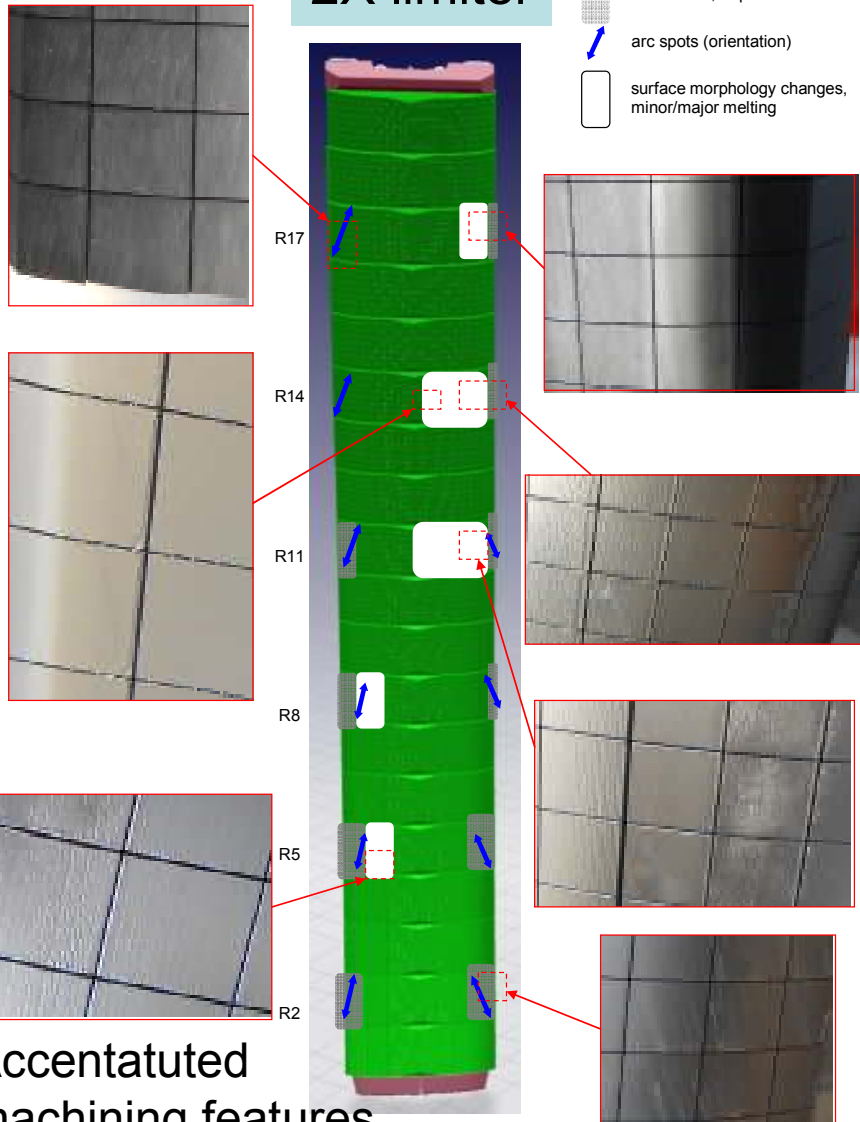
- Changes to tiles during the ILW campaigns
- First erosion and deposition data on tiles removed
 - Inner Wall Guard limiters
 - Dump plates
 - Outer Poloidal Limiters
 - Divertor tiles
 - Diagnostics in remote areas



Arcing



2X limiter

-  dark areas, deposition
-  arc spots (orientation)
-  surface morphology changes, minor/major melting

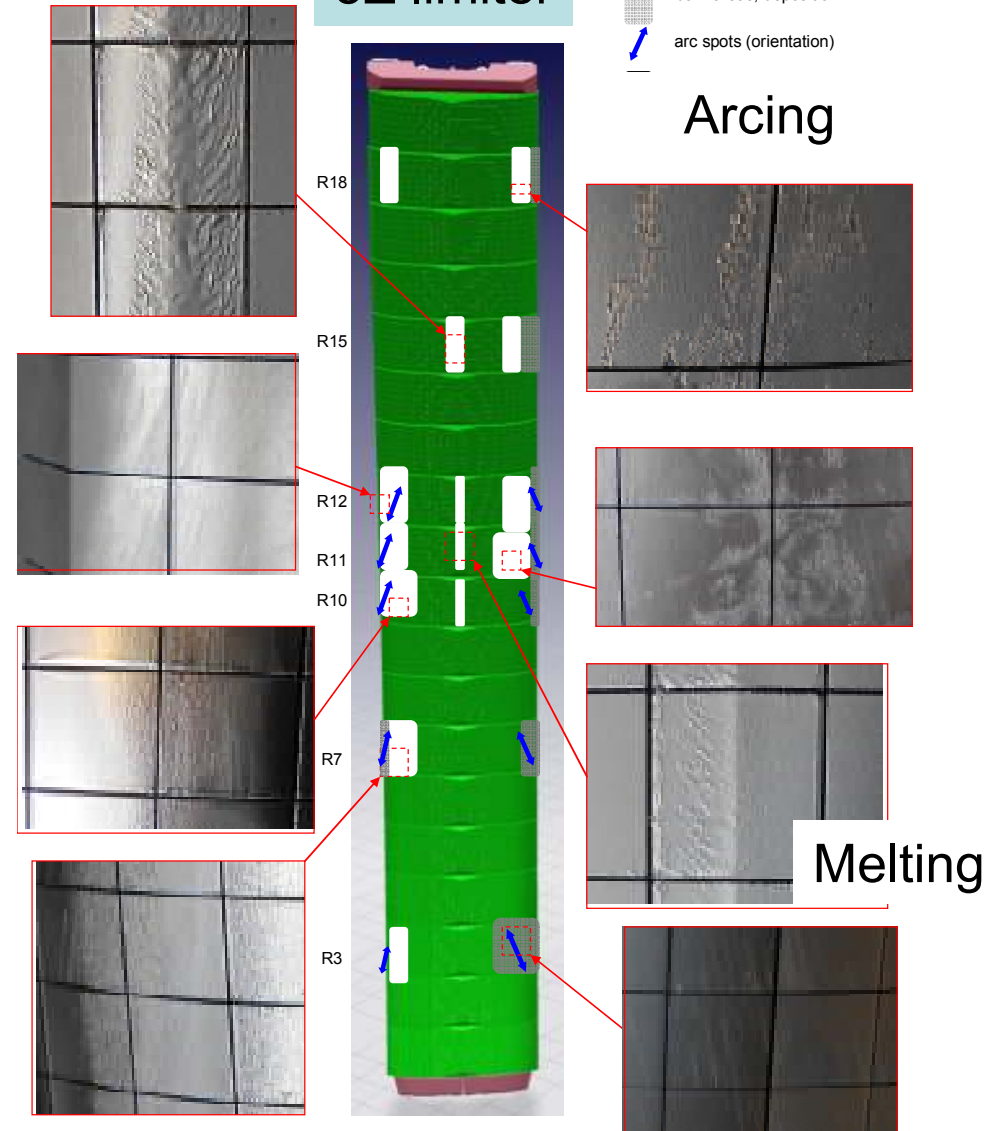


Accentuated machining features

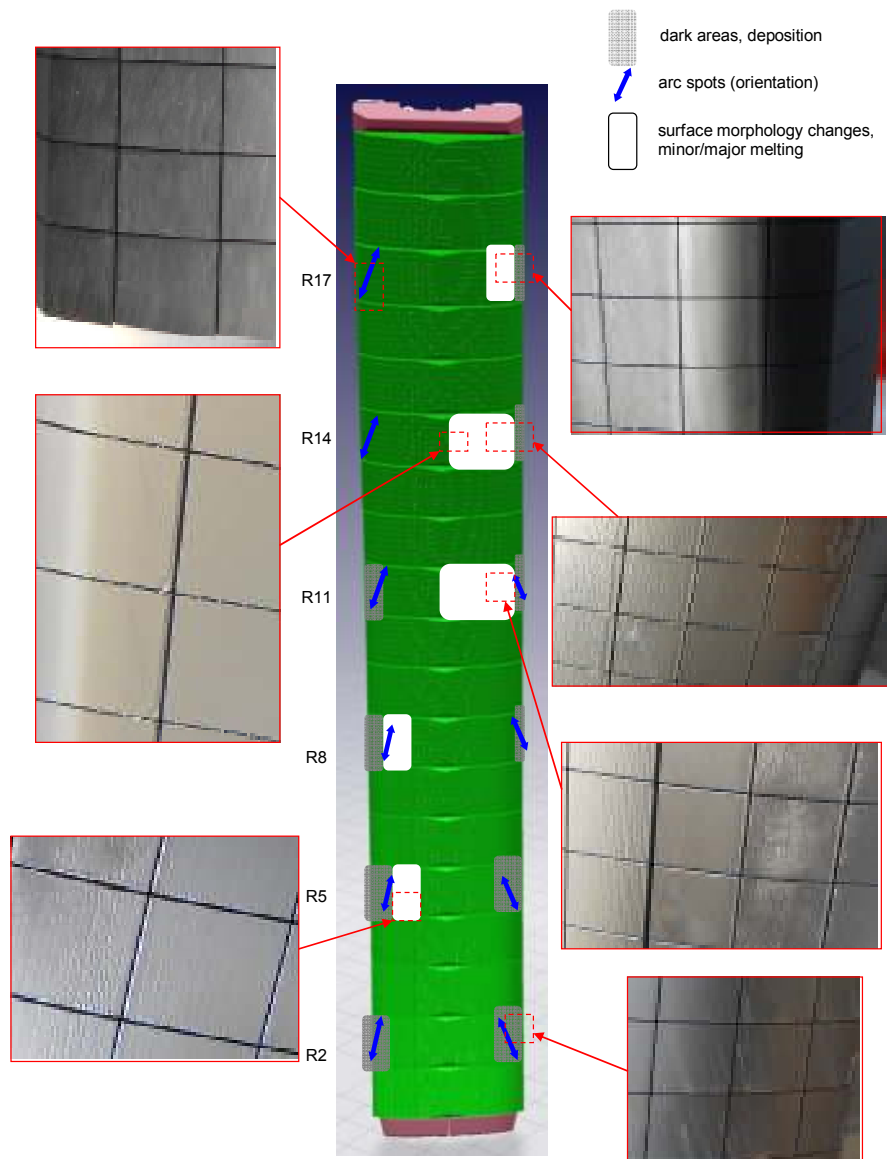
5Z limiter

-  dark areas, deposition
-  arc spots (orientation)

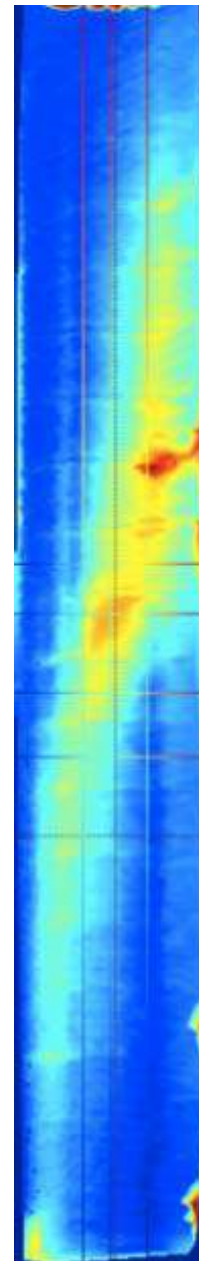
Arcing



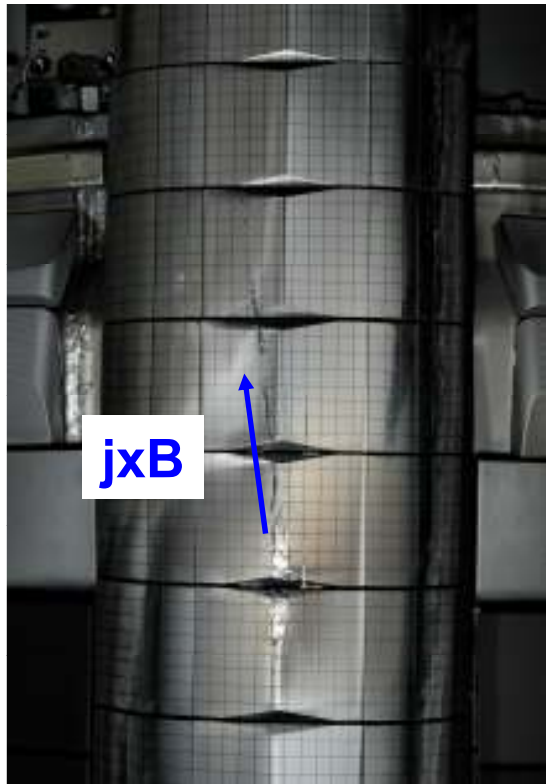
Melting



- Melting occurs during limiter phase
- More limiter only plasmas to test the wall



Heat Flux Distribution
Arnoux: PPMC14 I-14



4X Limiter

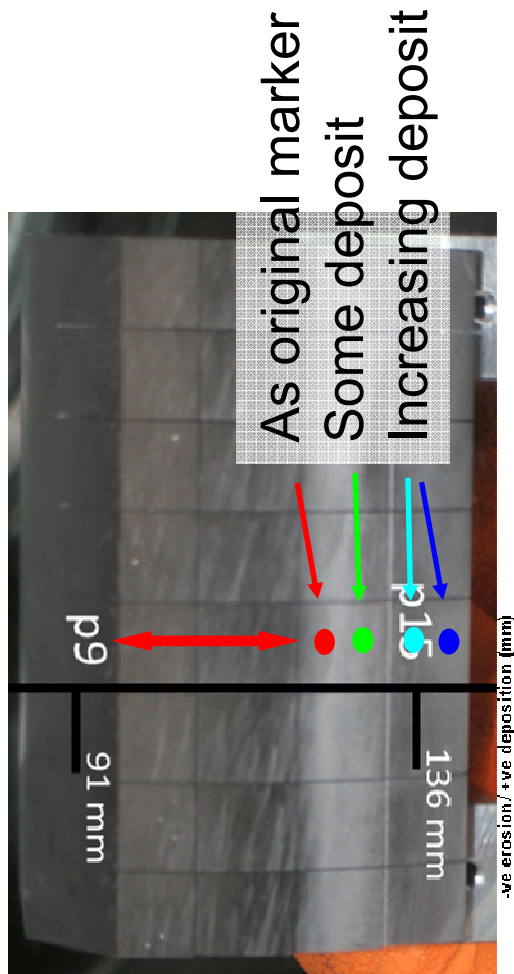


8X Limiter

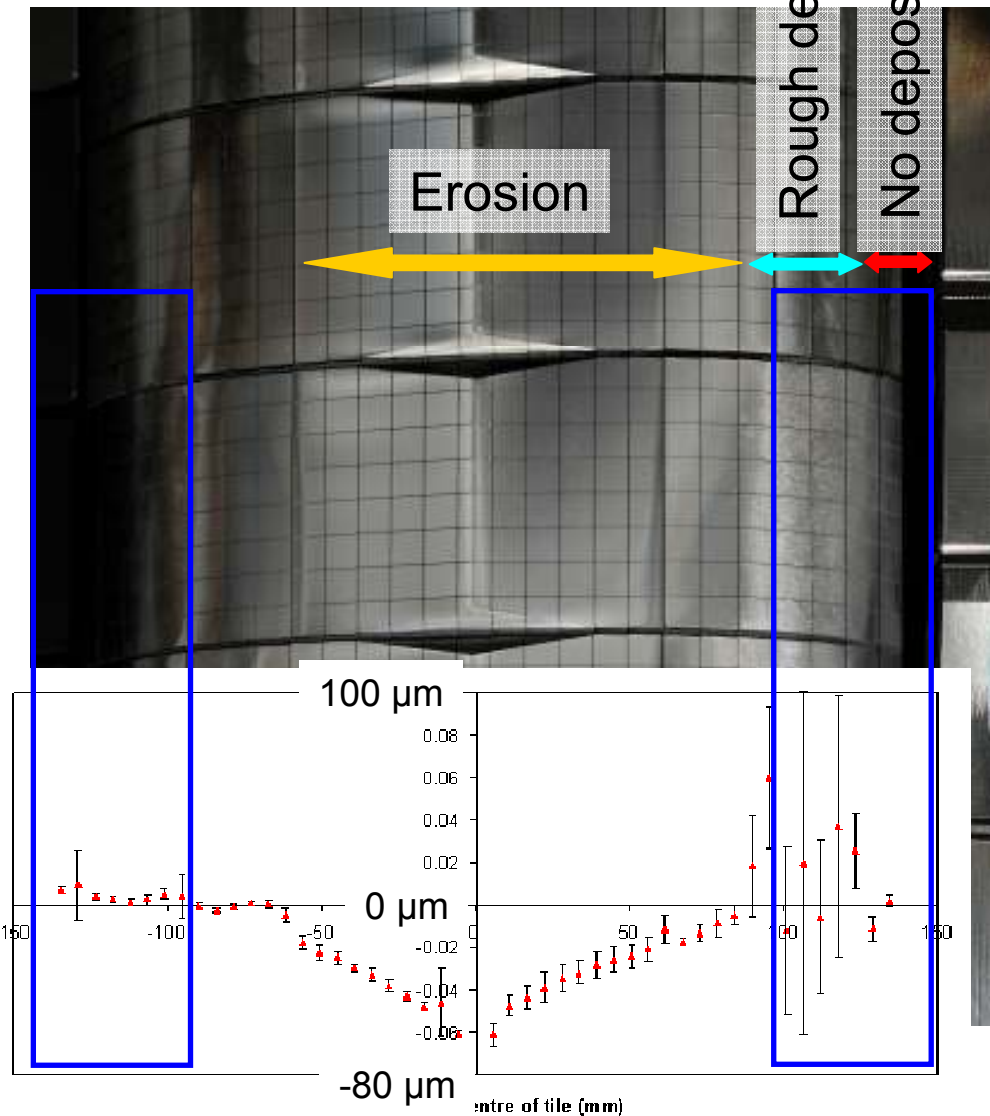
- Strong melting on two limiters during one pulse to test Be wall
- The melt position is consistent with the contact point
- The melt material driven up by Lorentz $j \times B$ forces
- Melt flow can travel a considerable distance upwards even jumping across gaps
- Maximum temperature of 920°C was recorded at melting

Sergienko PFMC-14
Arnoux: PFMC-14 I-14

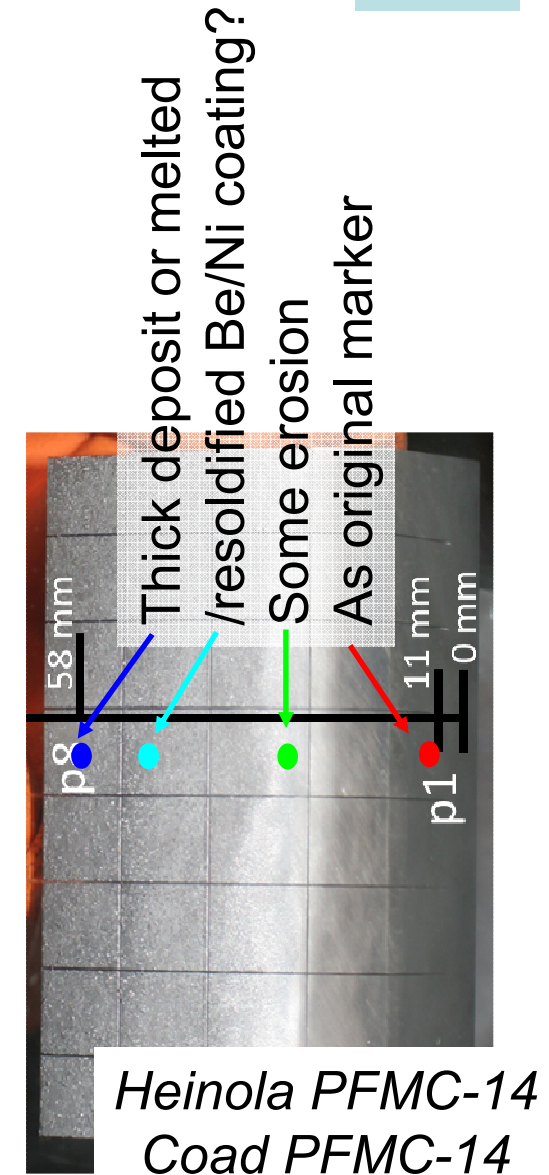
Ion Beam Analysis



SURFACE PROFILING

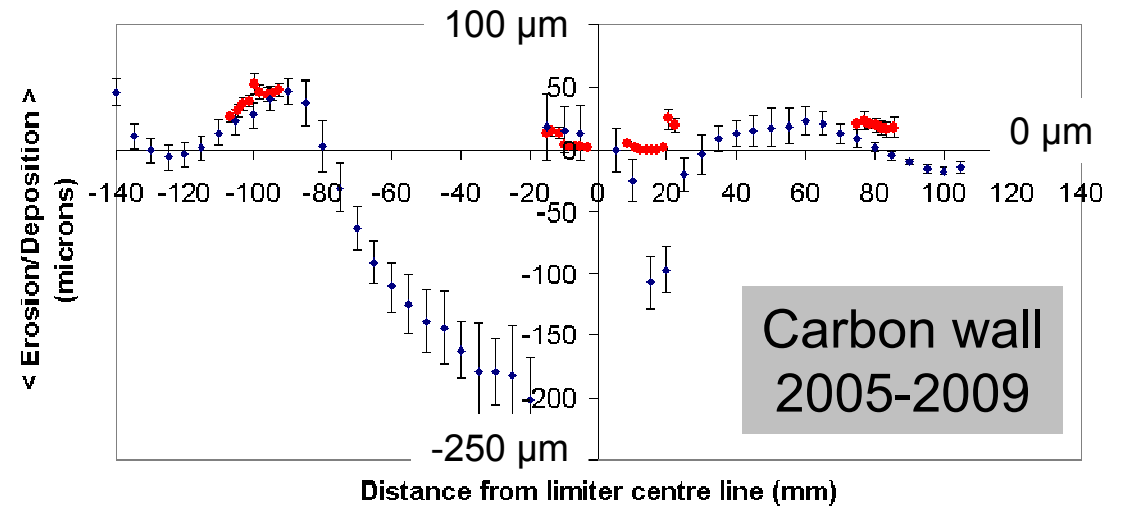
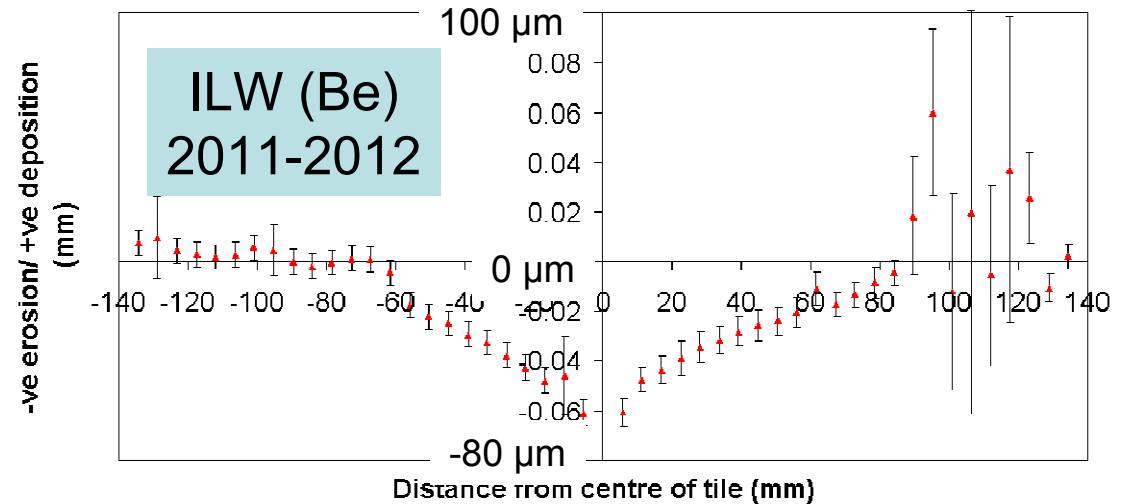
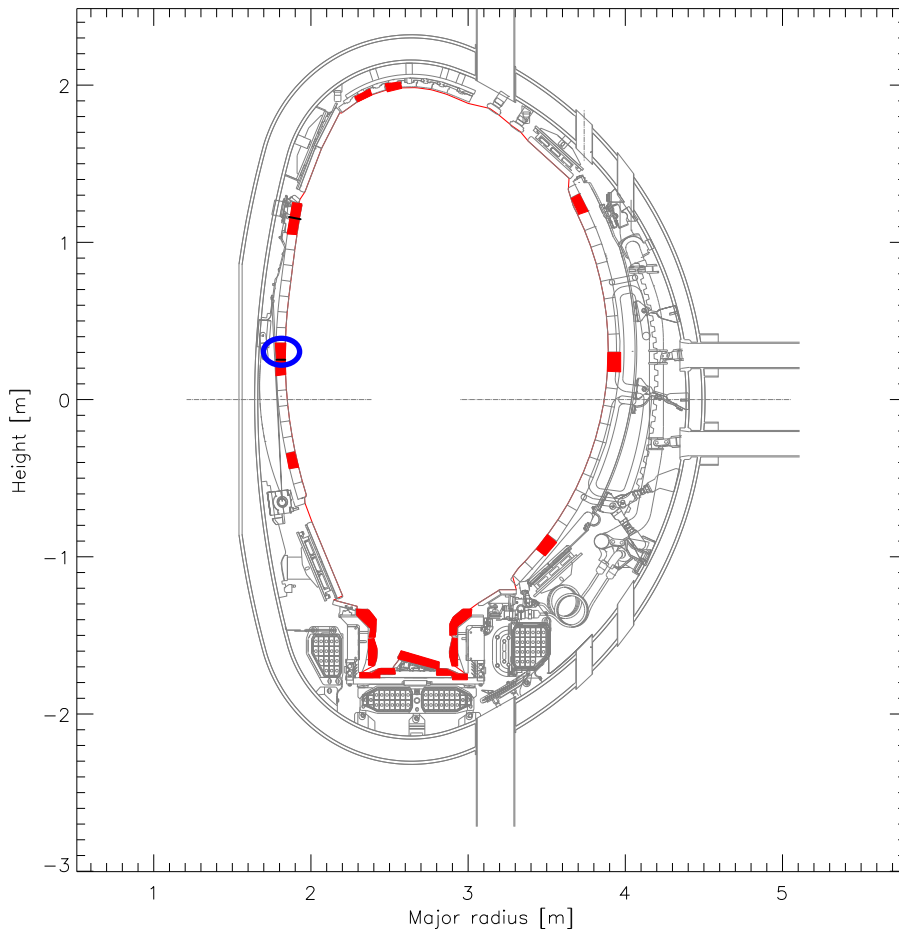


IBA





Comparison of erosion and deposition on mid-plane IWGL tile



Heinola PFMC-14

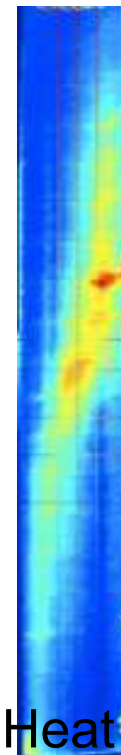
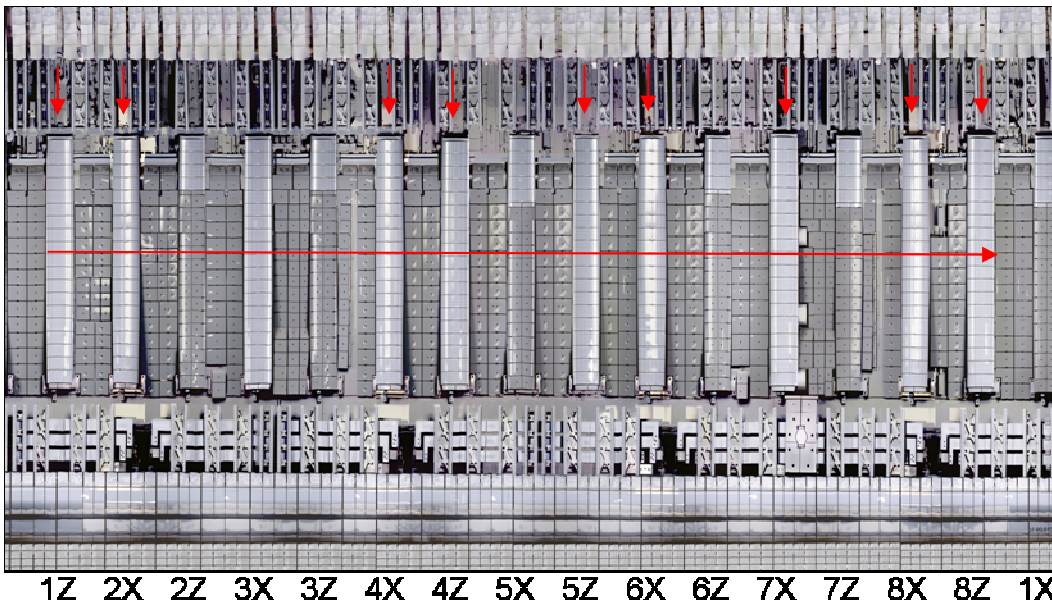
Net erosion from mid-plane IWGL tile Beryllium vs Carbon

	Net erosion rate (Atoms/s)
2005-2009 (C)	1.4e19
2011-2012 (Be)	2.3e19

Net erosion rate from one row of mid-plane IWGL tiles is higher for the beryllium wall than carbon wall

- 16 limiters for carbon wall
- 10 limiters for beryllium wall

Further analysis required to fully assess main chamber erosion and deposition

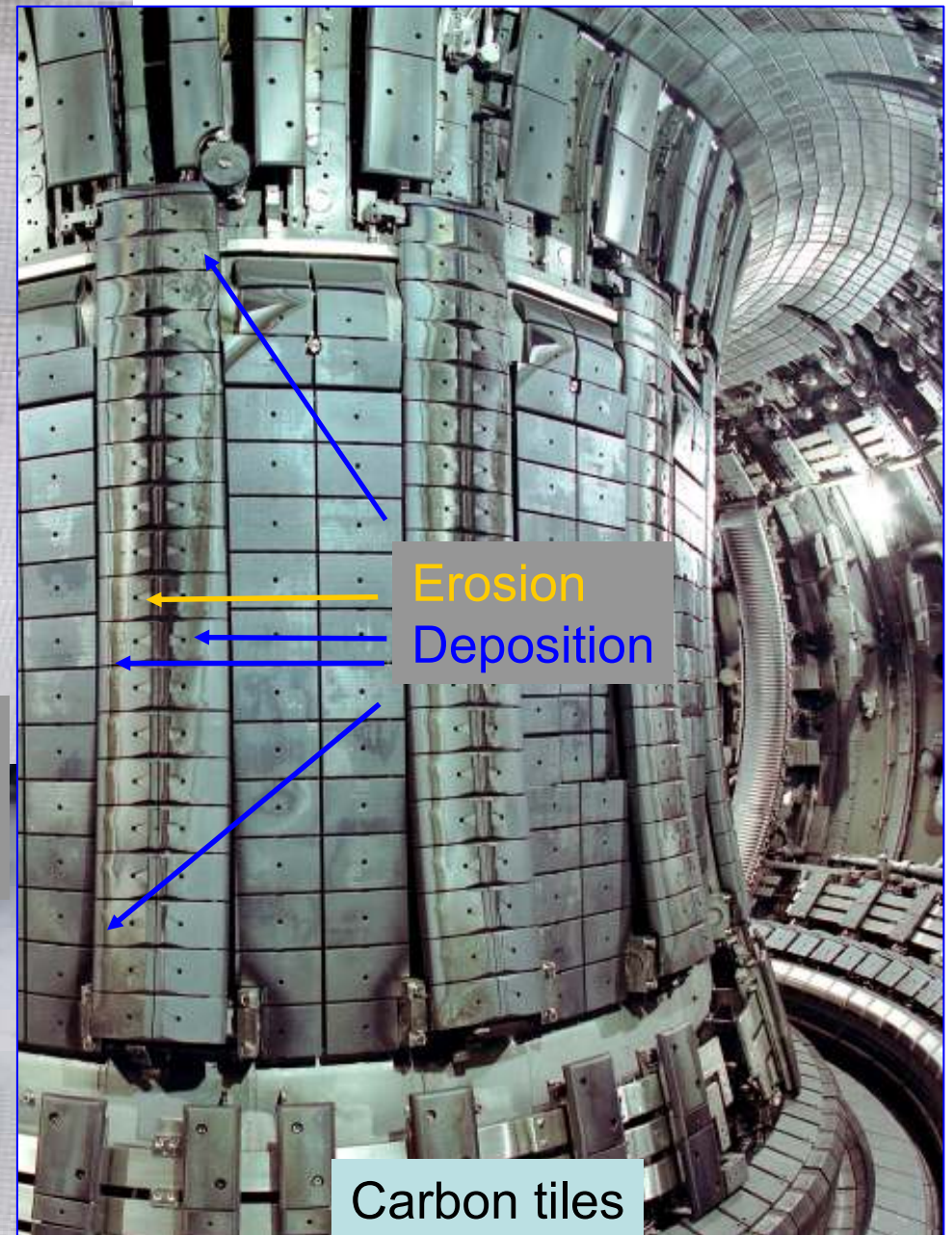
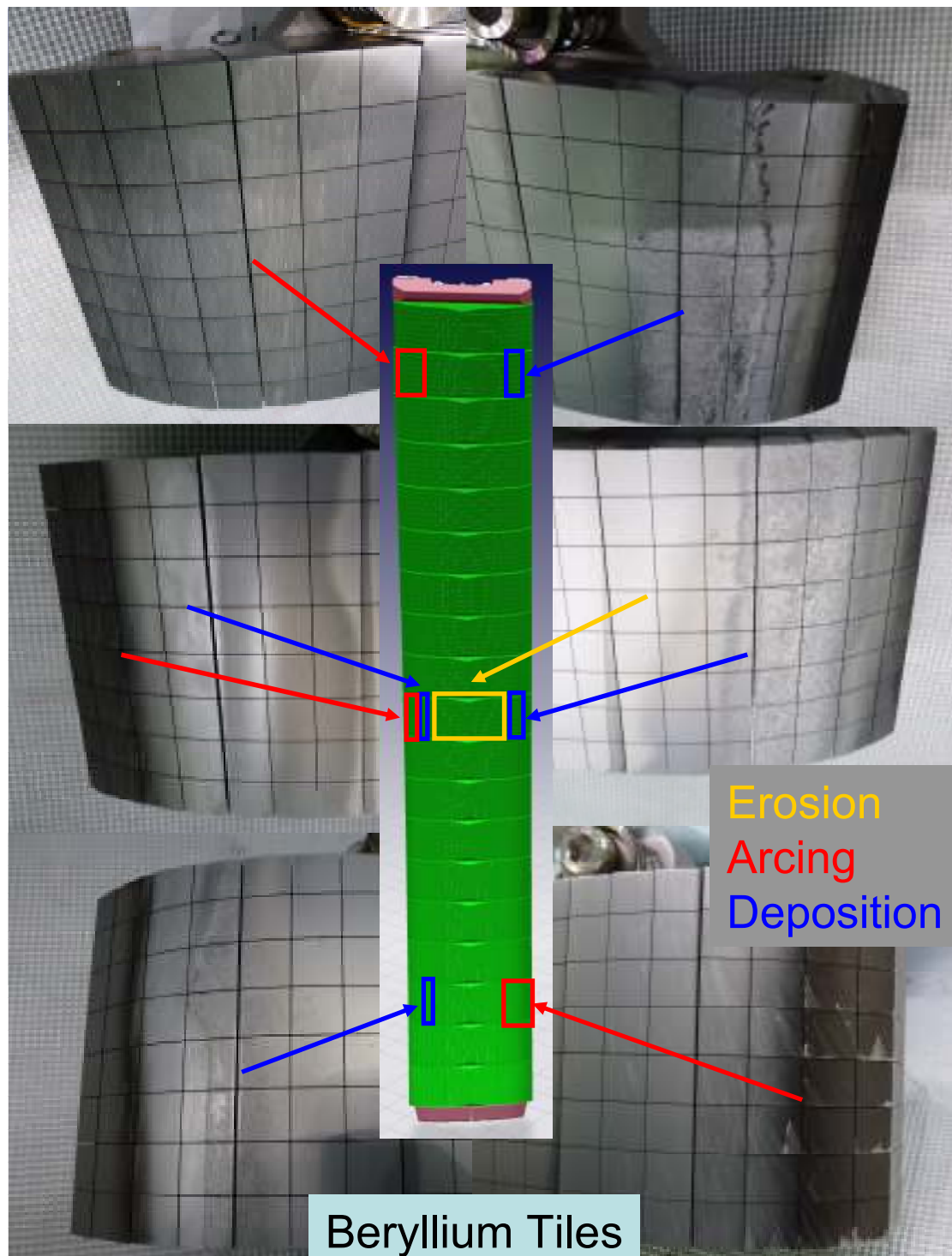


Plasma times

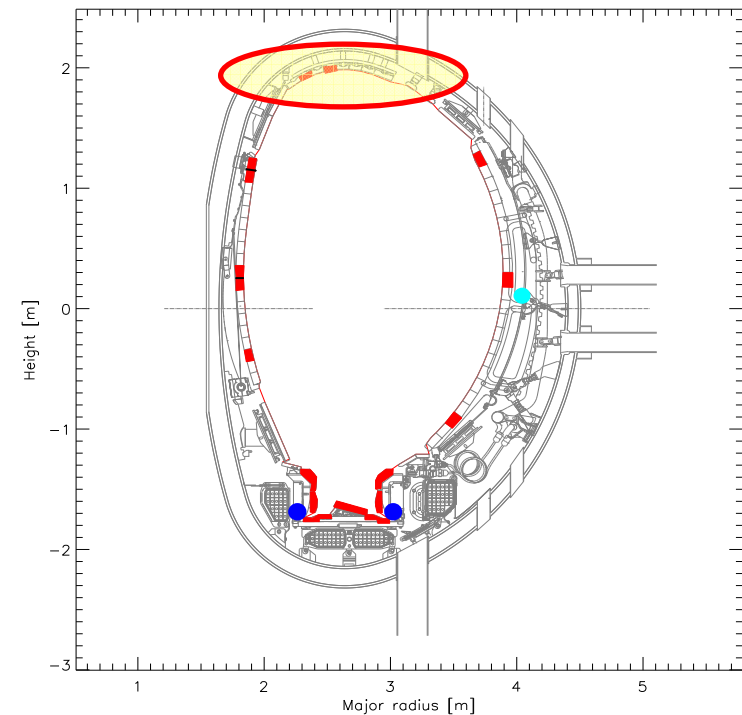
	Total plasma time (h)	X-point (h)	Limiter (h)
2005-2007	35	27	8
2008-2009	45	33	12
2011-2012	19	13	6

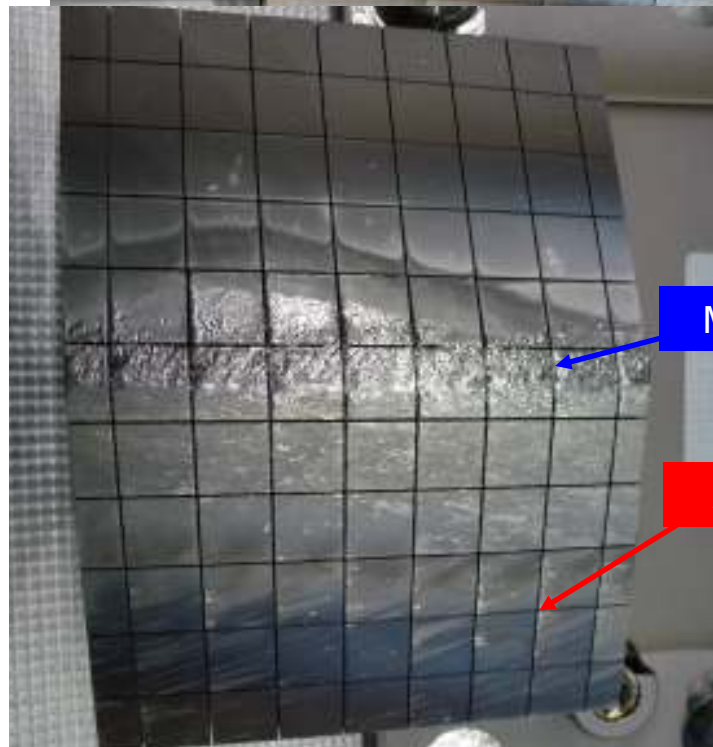
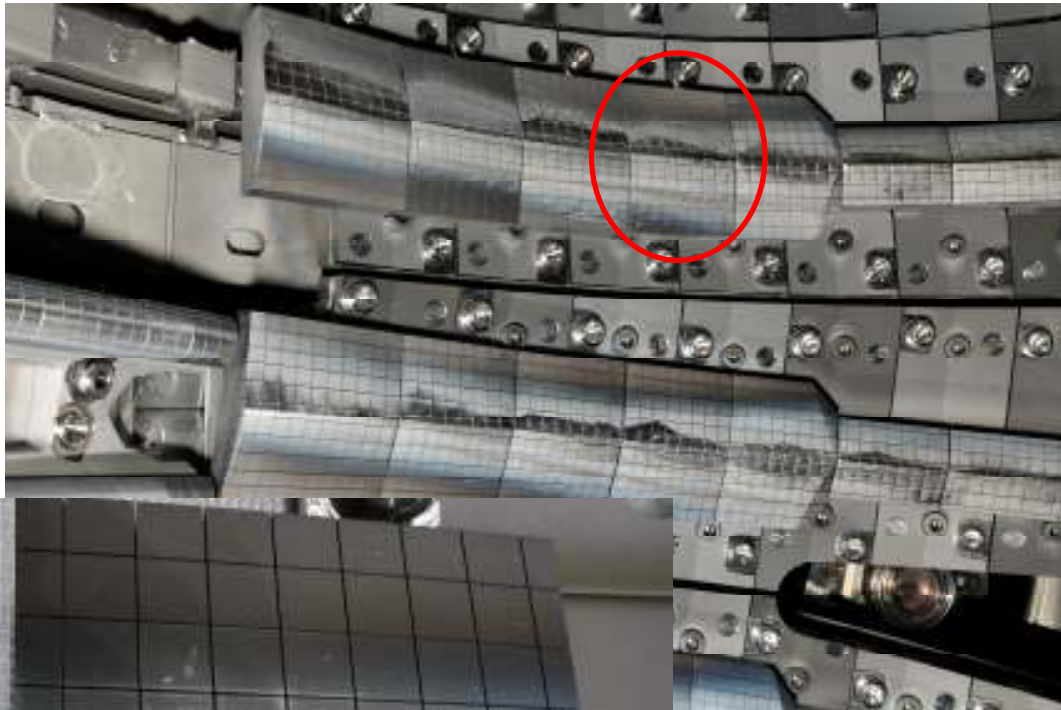
Heat Flux Distribution *Arnoux PFMC-14*

IWGL Erosion/Deposition: Beryllium vs Carbon



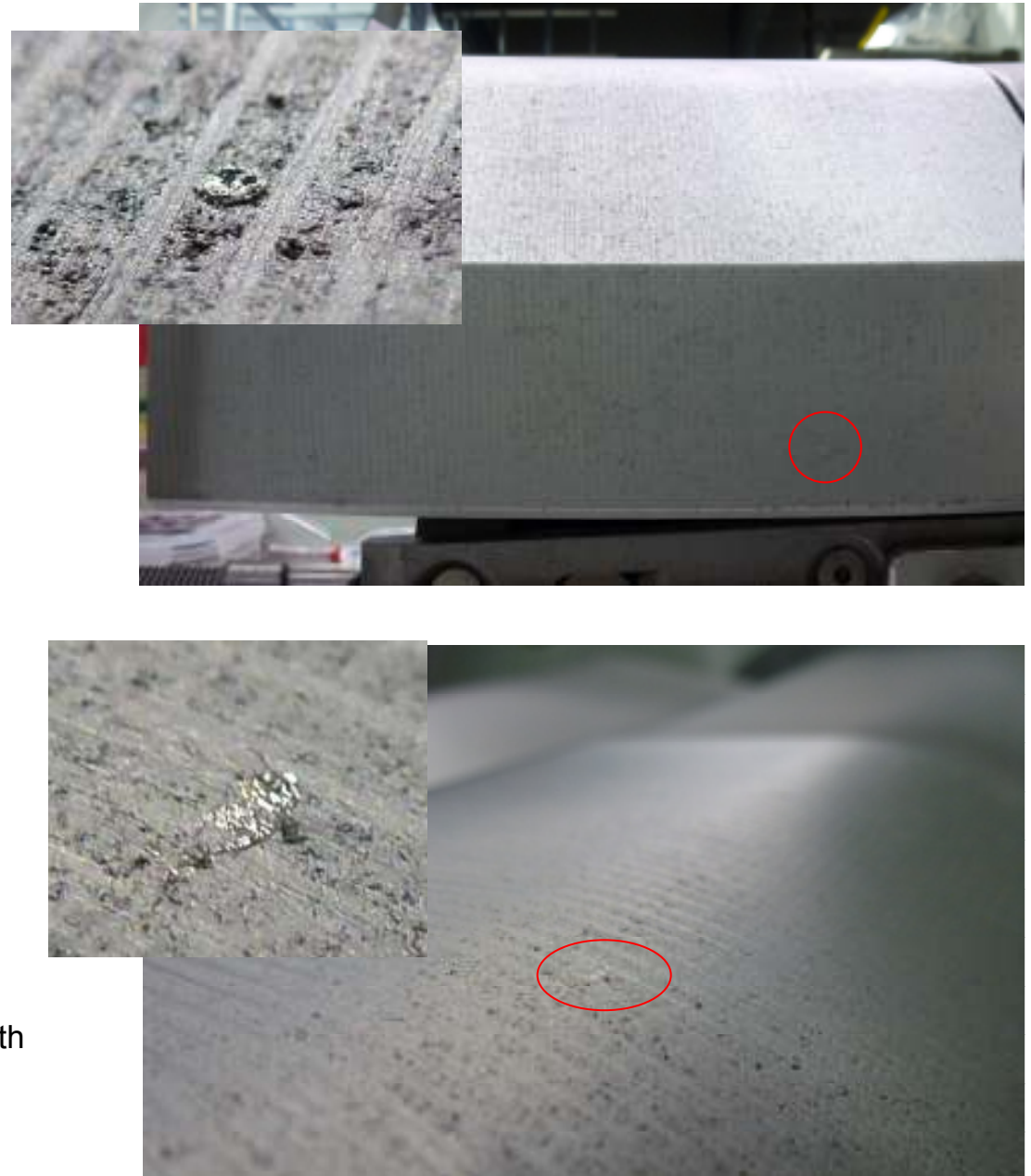
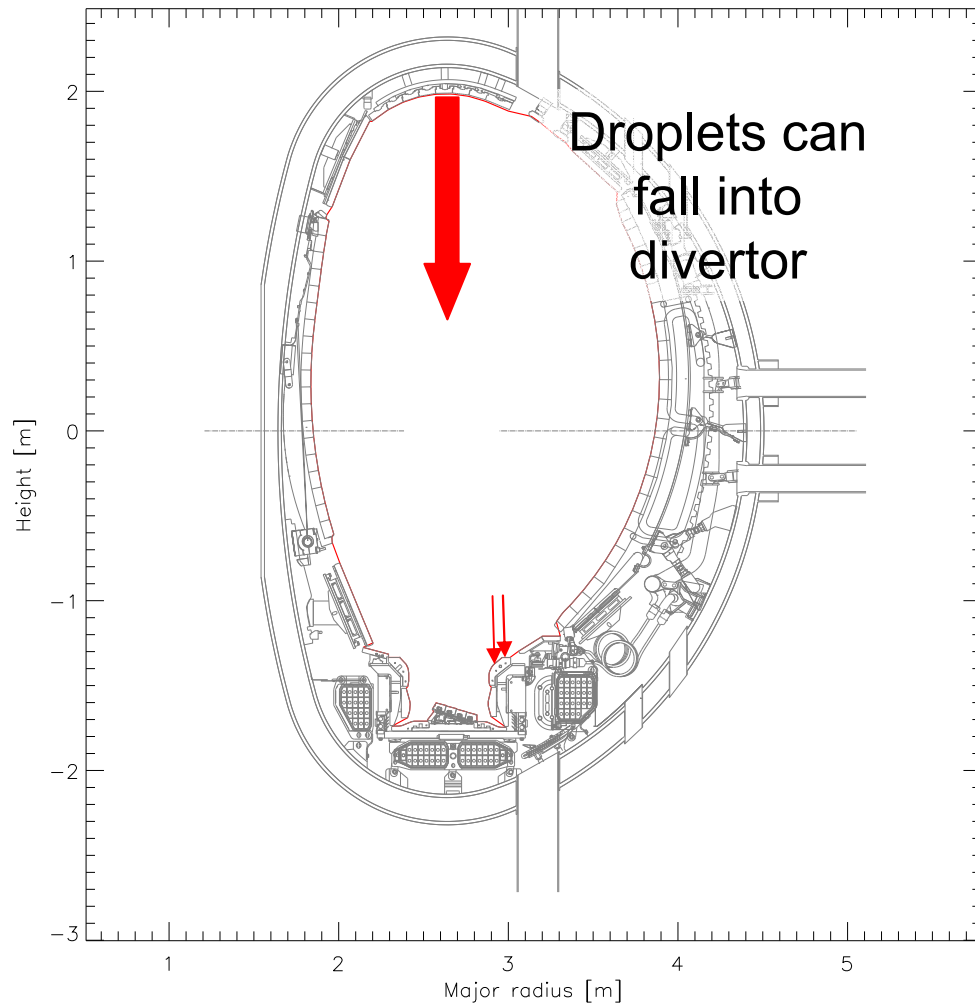
- Changes to tiles during the ILW campaigns
- First erosion and deposition data on tiles removed
 - Inner Wall Guard limiters
 - **Dump plates**
 - Outer Poloidal Limiters
 - Divertor tiles
 - Diagnostics in remote areas





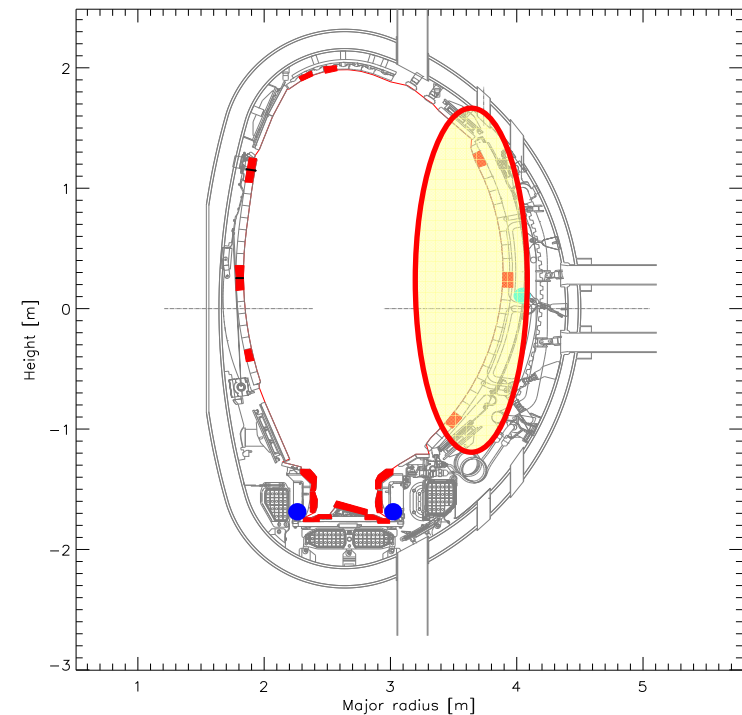
- Melt damage occurred prior to use of the disruption mitigation valve
- Toroidally uniform
- Additional source of beryllium erosion

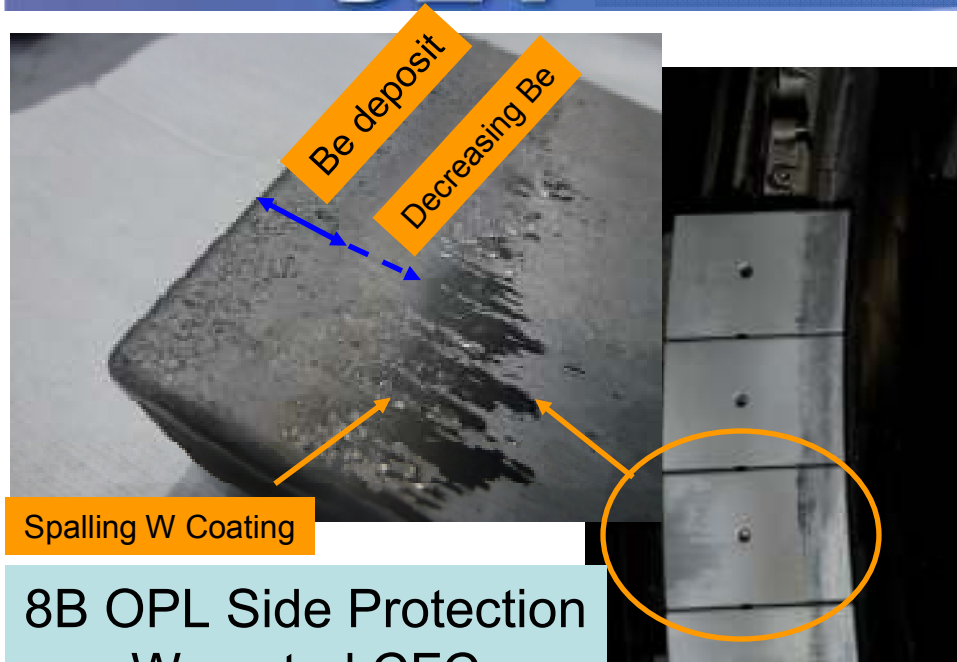
*Heinola PFMC-14
Arnoux PFMC-14 I-14*



<10 beryllium droplets observed on 1/12th
divertor tiles closely examined

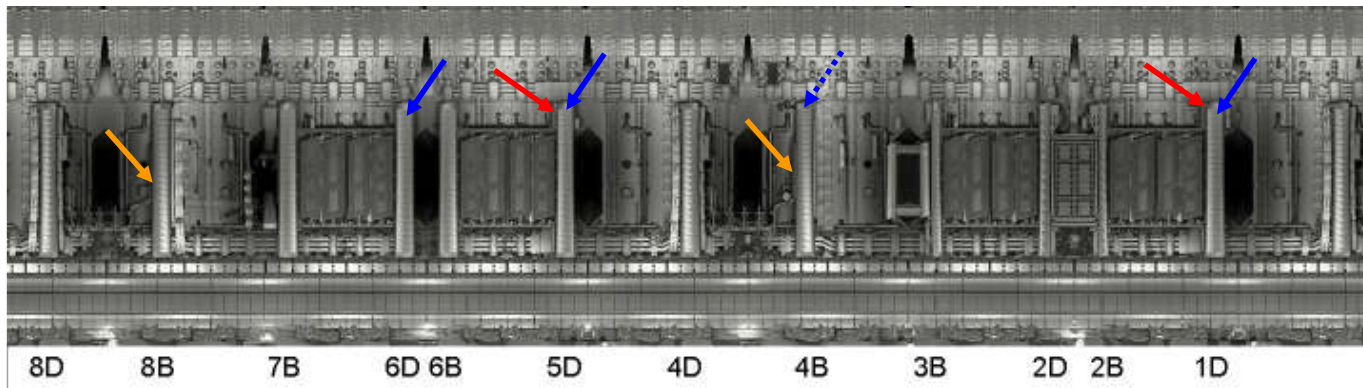
- Changes to tiles during the ILW campaigns
- First erosion and deposition data on tiles removed
 - Inner Wall Guard limiters
 - Dump plates
 - **Outer Poloidal Limiters**
 - Divertor tiles
 - Diagnostics in remote areas



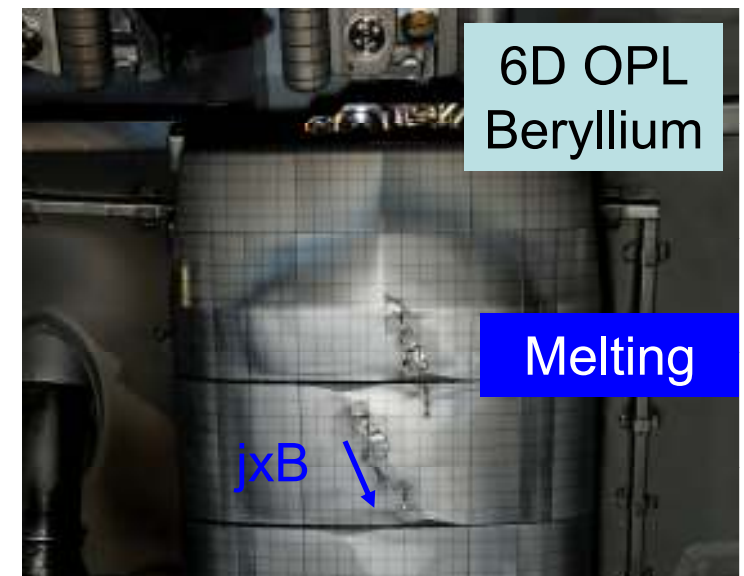
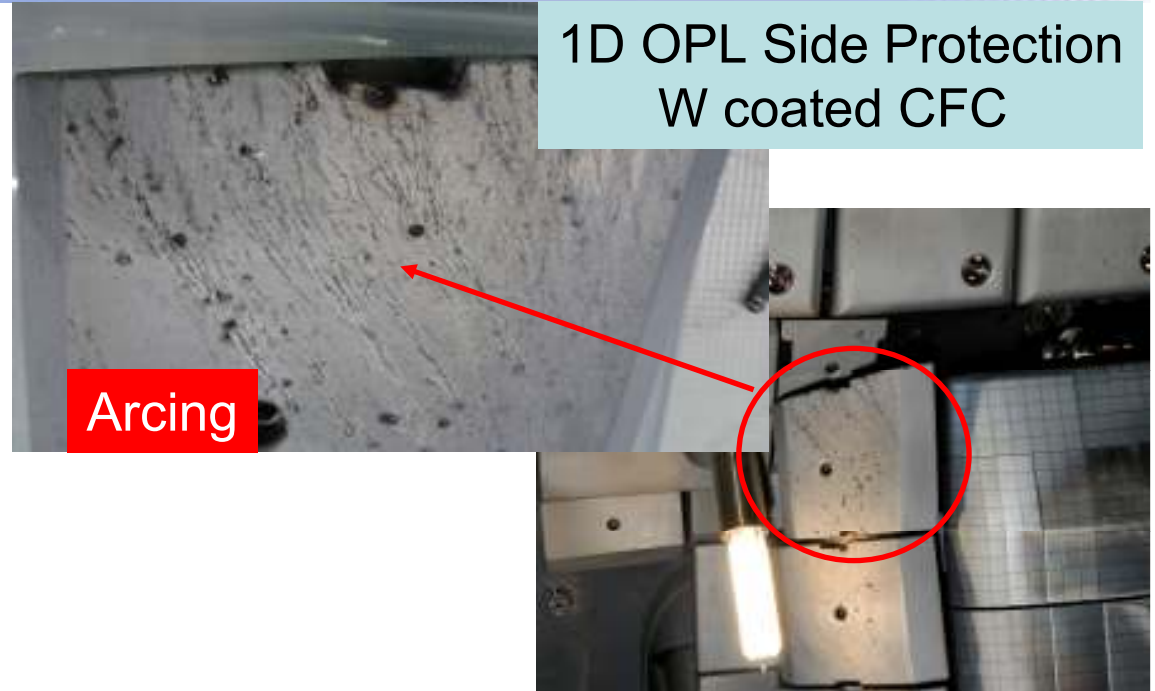


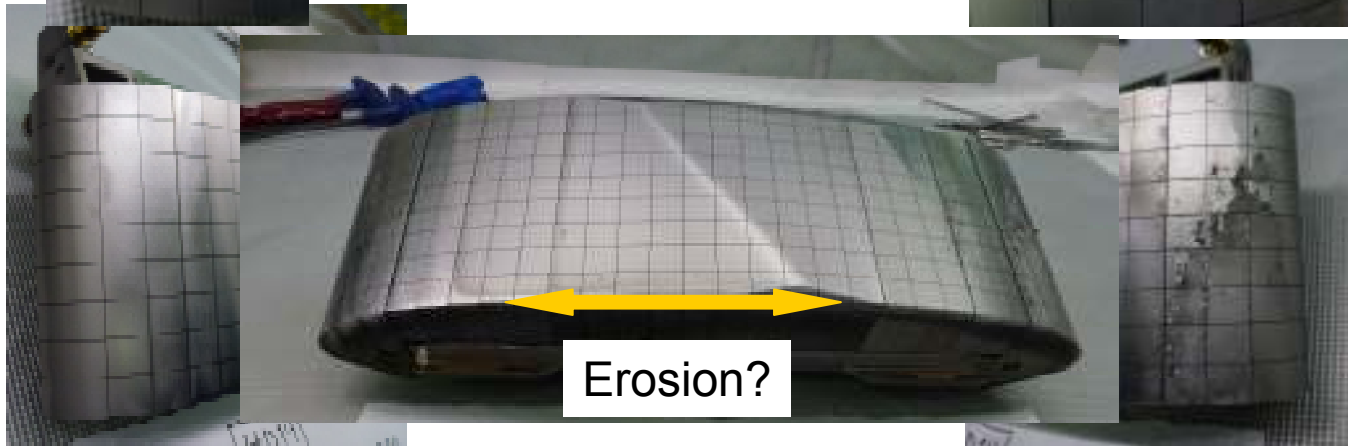
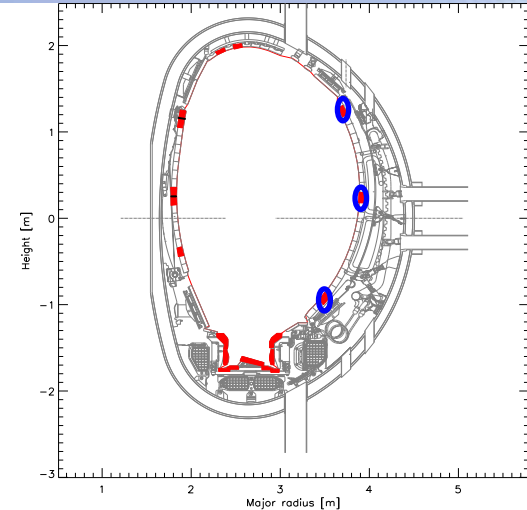
8B OPL Side Protection
W coated CFC

Example of Be deposition in the main chamber

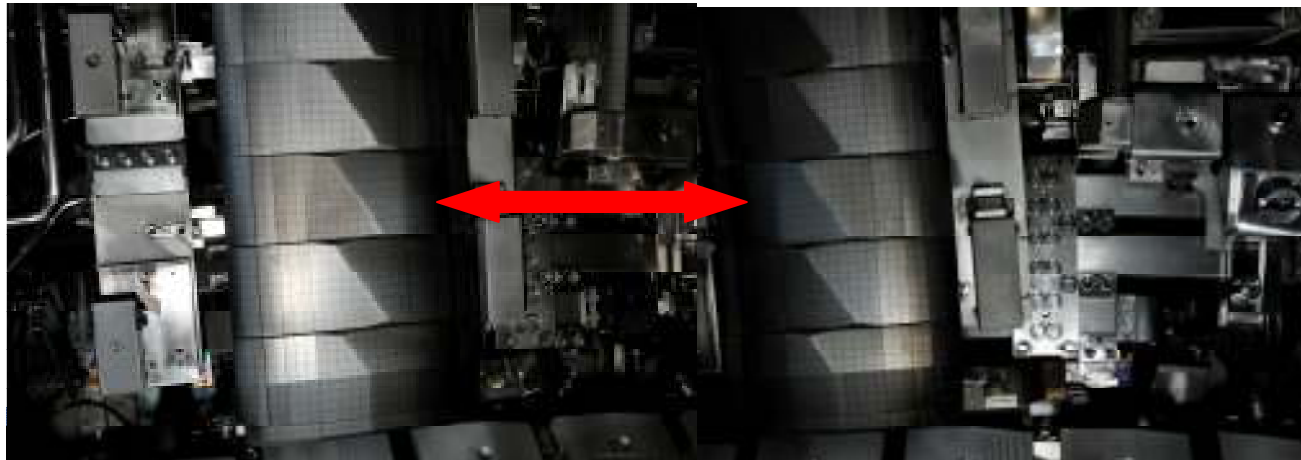


Ruset PFMC-14, Sergienko PFMC-14

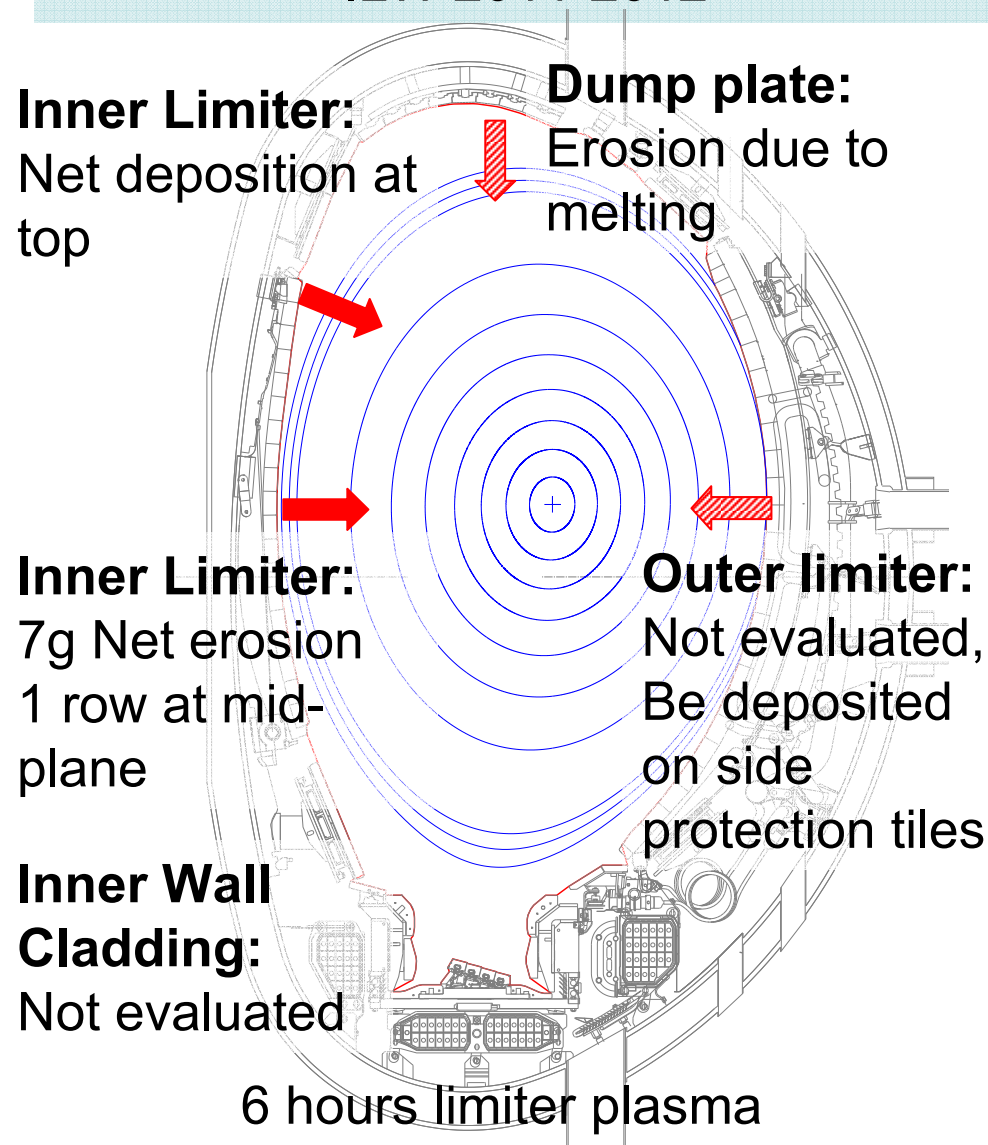




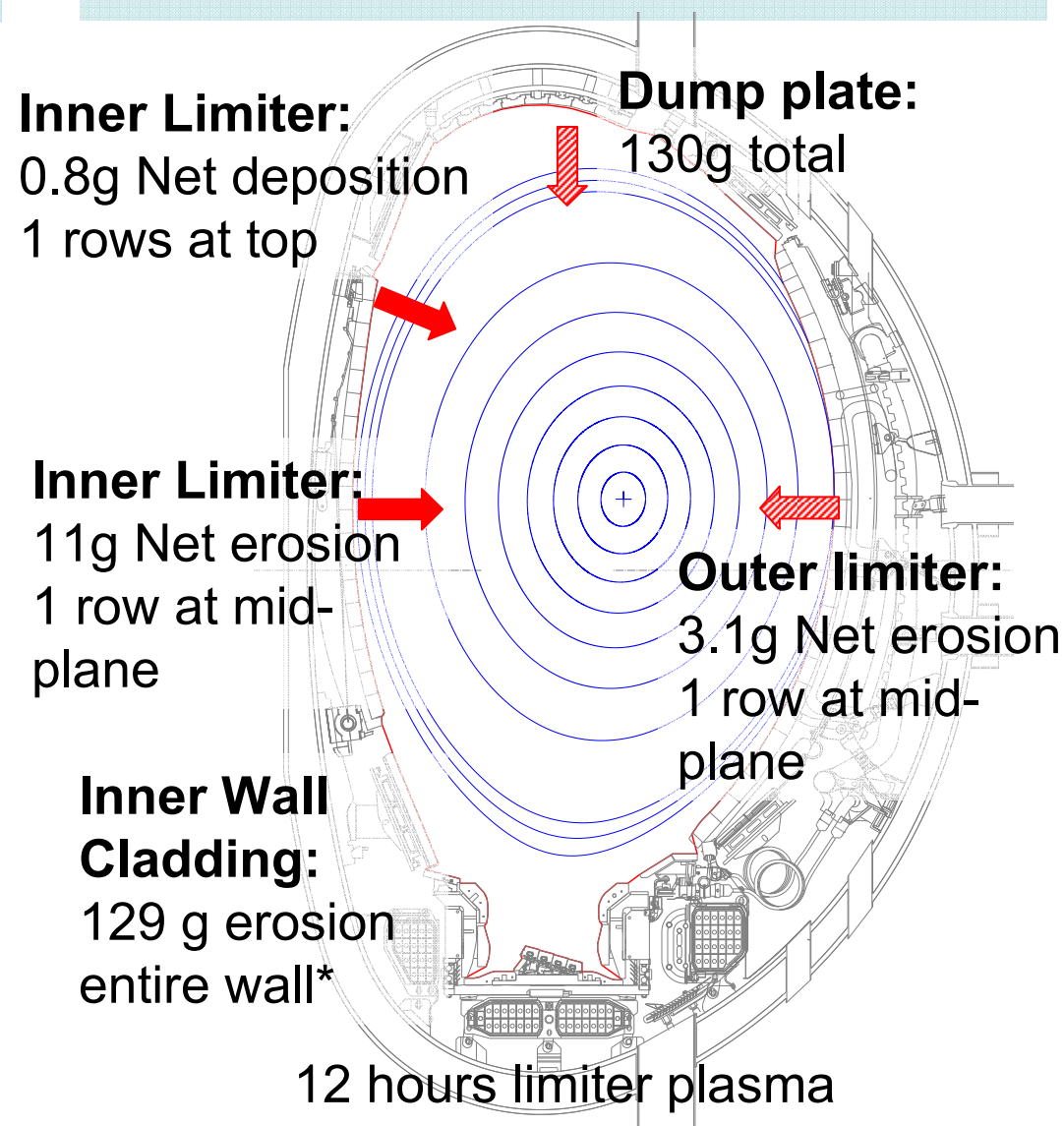
- 3 OPL tiles taken out for post mortem analysis
- Local deposition at outer ends of tile (also on side protection tiles)
- Possible erosion from centre of tile
- Spalling of marker coating at right hand end of tile
- Analysis of OPL tiles to be completed



ILW 2011-2012

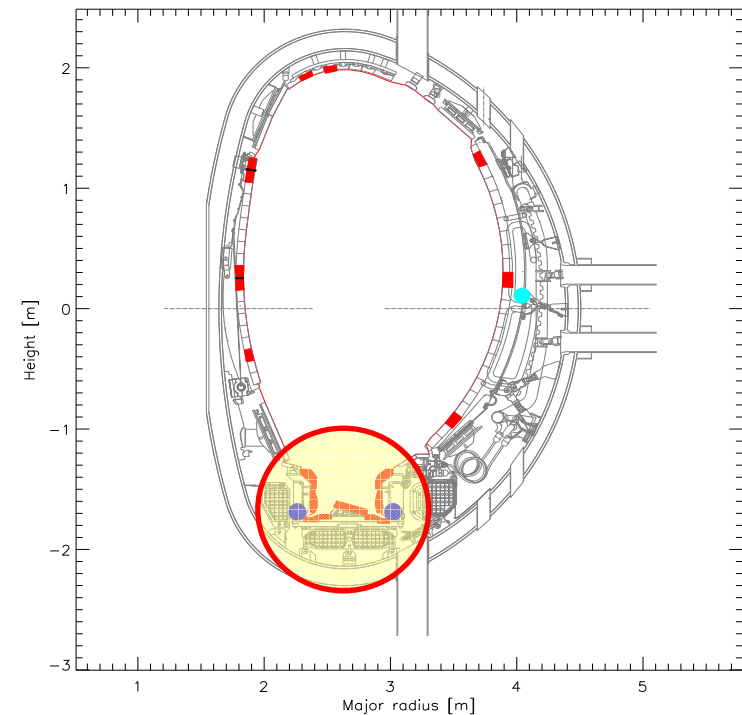


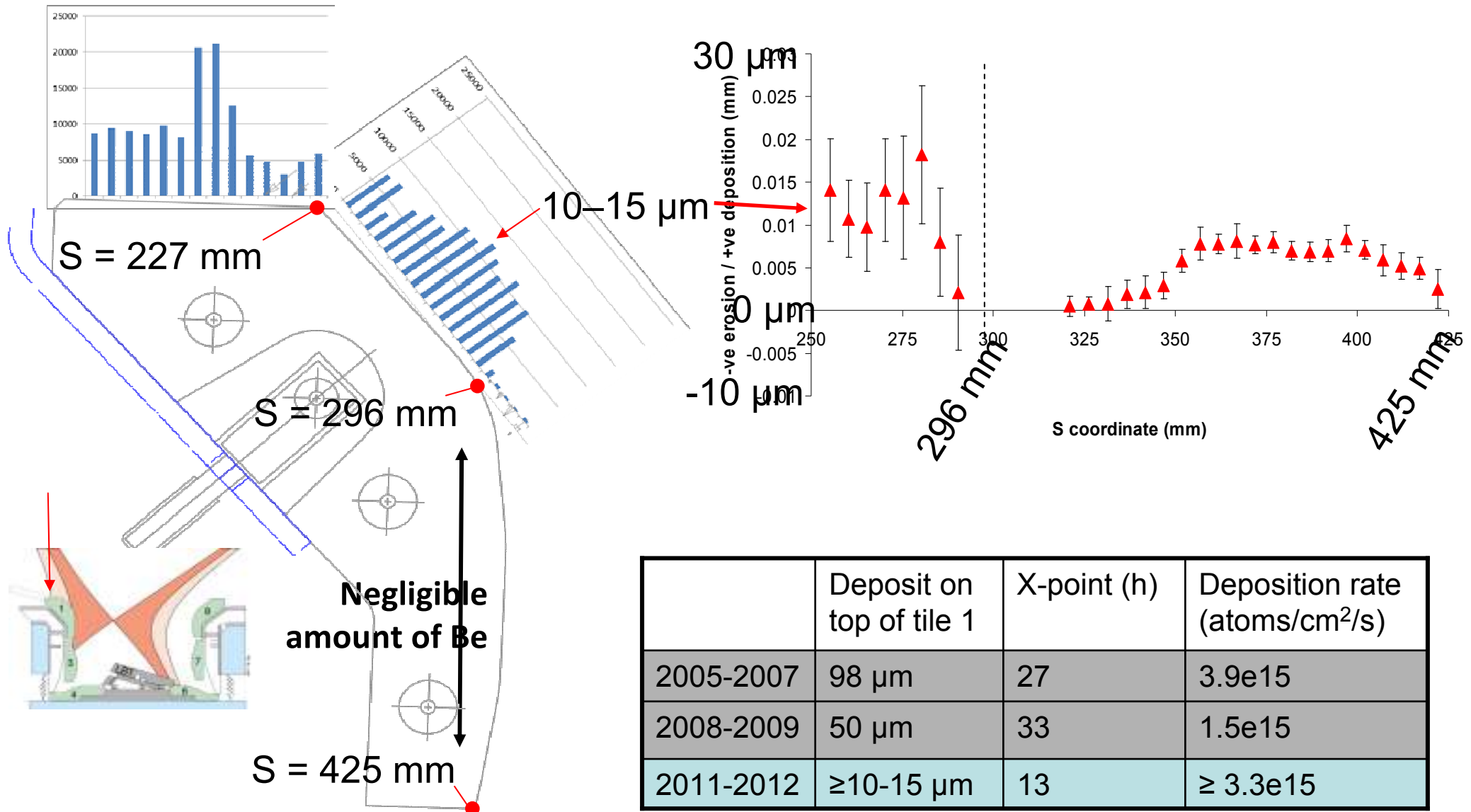
Carbon Wall 2008-2009



*Mayer <http://dx.doi.org/10.1016/j.jnucmat.2013.01.167>, Widdowson <http://dx.doi.org/10.1016/j.jnucmat.2013.01.179>

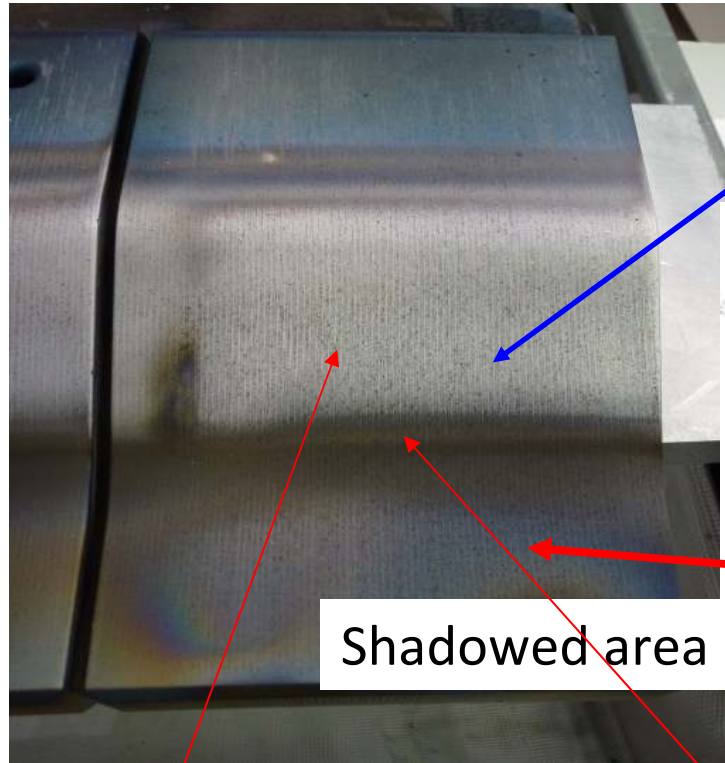
- Changes to tiles during the ILW campaigns
- First erosion and deposition data on tiles removed
 - Inner Wall Guard limiters
 - Dump plates
 - Outer Poloidal Limiters
 - **Divertor tiles**
 - Diagnostics in remote areas





Coad PFMC-14
Heinola PFMC-14

Inner divertor - Tile 4



Thin silvery deposit

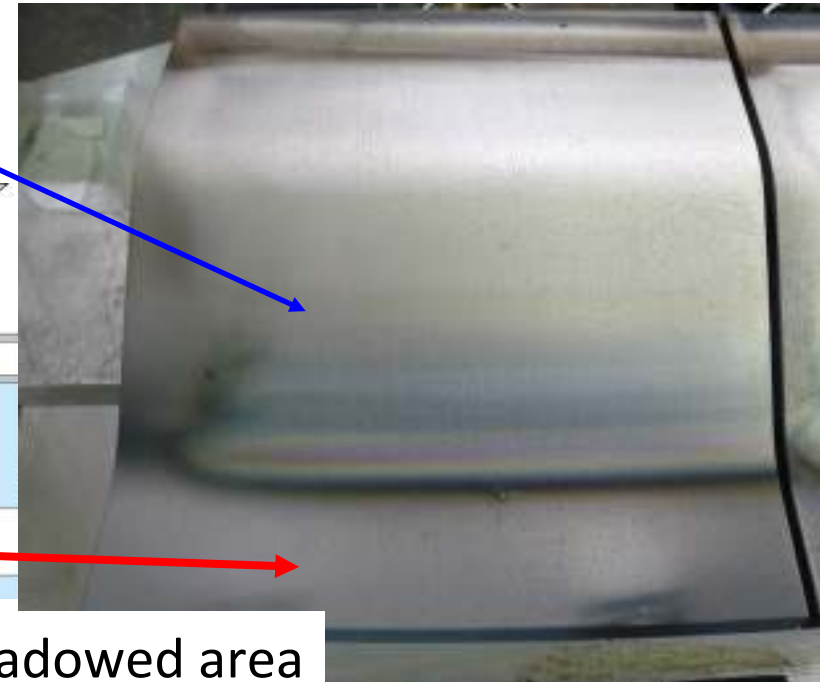
Shadowed area

$\text{Be} \sim 3 \times 10^{18} \text{ atoms cm}^{-2}$
(~250nm)

$\text{Be} \sim 6 \times 10^{17} \text{ atoms cm}^{-2}$
(~50nm)

Heinola PFMC-14
Coad PFMC-14

Outer divertor - Tile 6

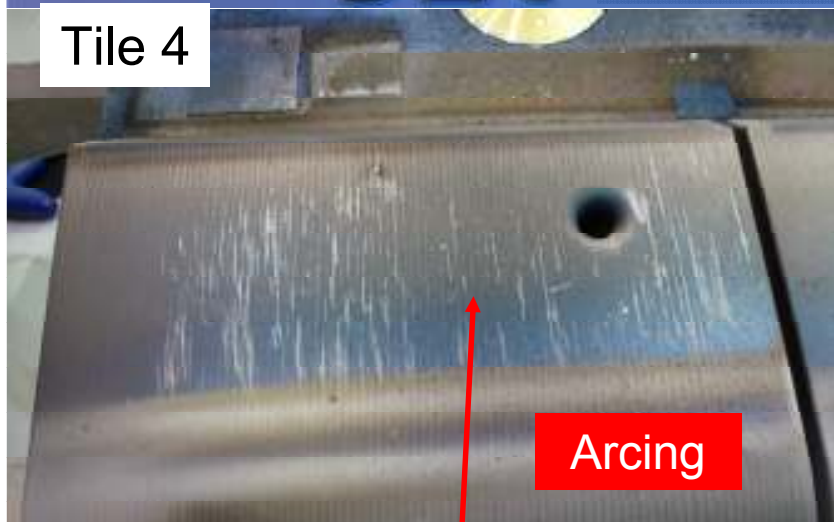


Shadowed area

- Thin deposits have formed at the inner and outer divertor corner tiles
- No erosion/deposition observed from surface profile measurements



Tile 4



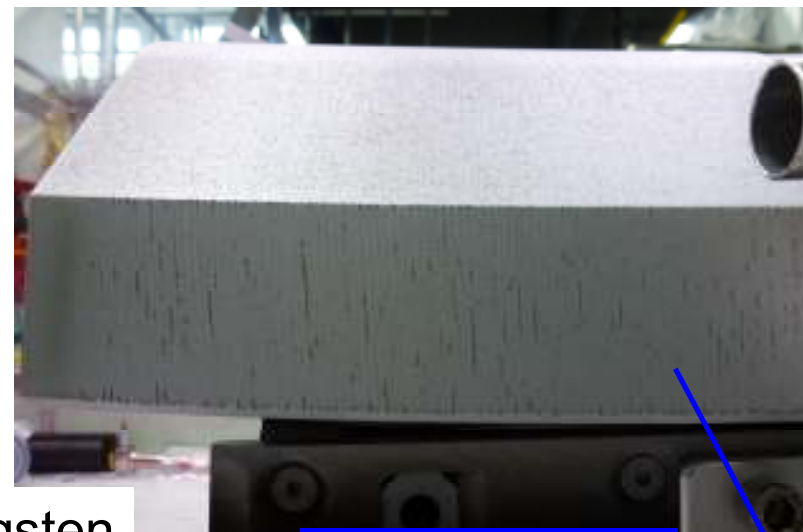
- Tungsten coatings are largely intact
- Arcing on tile 4 around whole vessel
- Delamination on tile 8 probably due to fibre orientation on top surface

Ruset PFMC-14

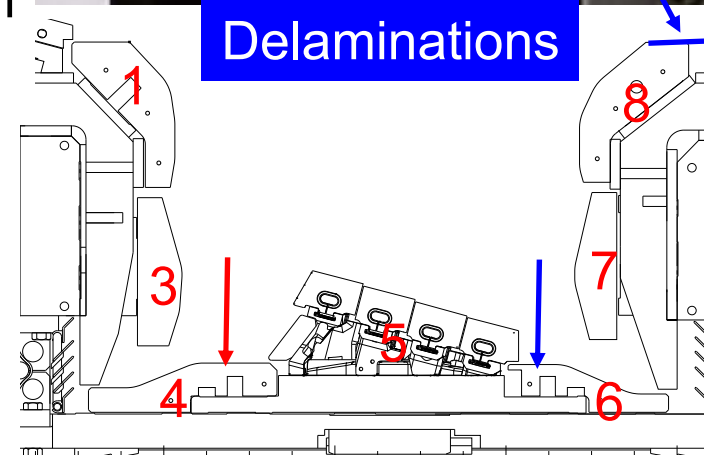
Outer



Inner



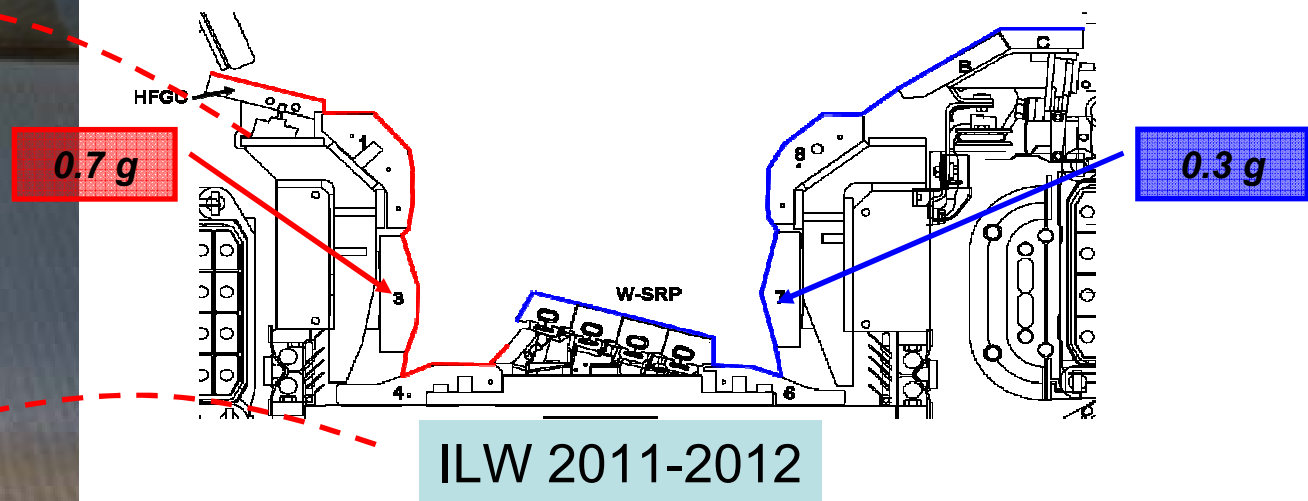
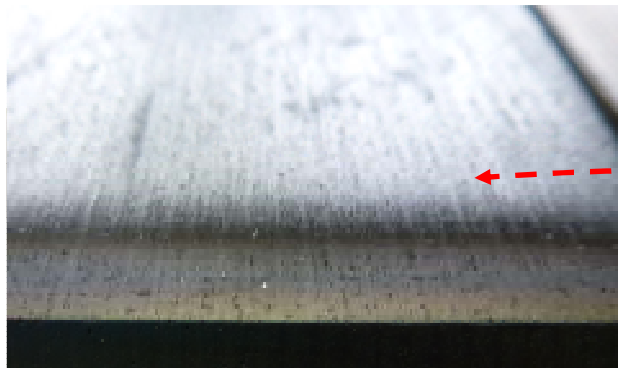
Delaminations



Dust

- Main source of dust for carbon wall was spalling deposits
- No thick deposits in divertor for ILW
- Continue to monitor dust production and layer growth in future interventions

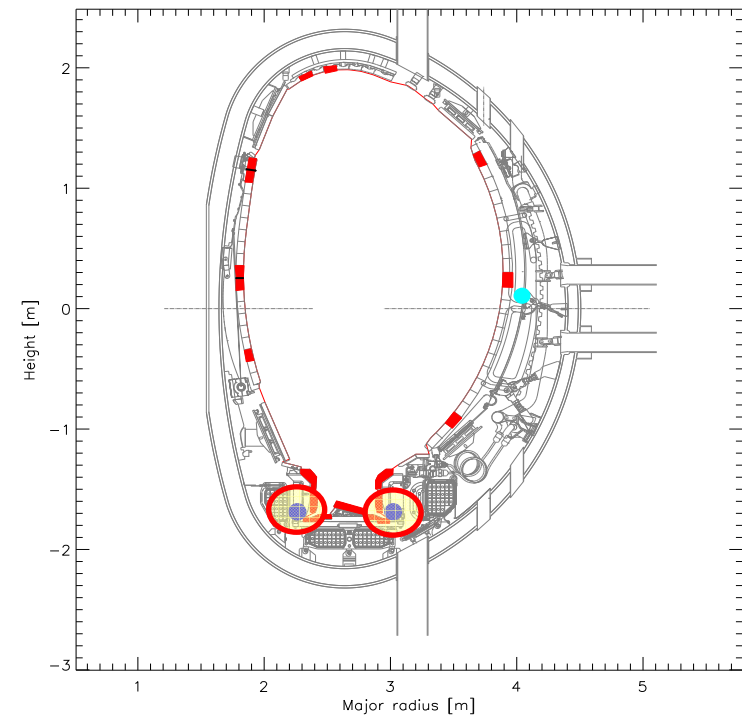
Carbon Wall 2008-2009



ILW 2011-2012

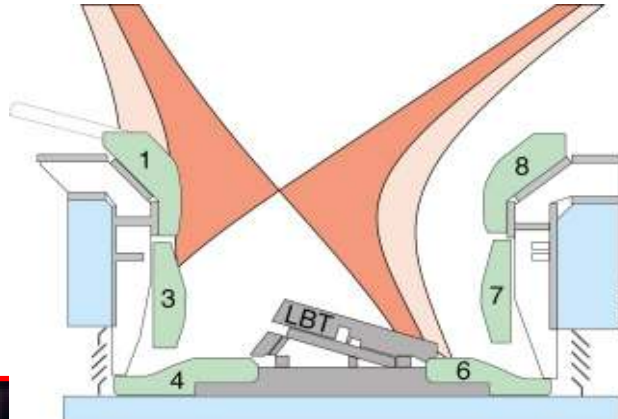
Heinola PFMC-14

- Changes to tiles during the ILW campaigns
- First erosion and deposition data on tiles removed
 - Inner Wall Guard limiters
 - Dump plates
 - Outer Poloidal Limiters
 - Divertor tiles
 - **Diagnostics in remote areas**



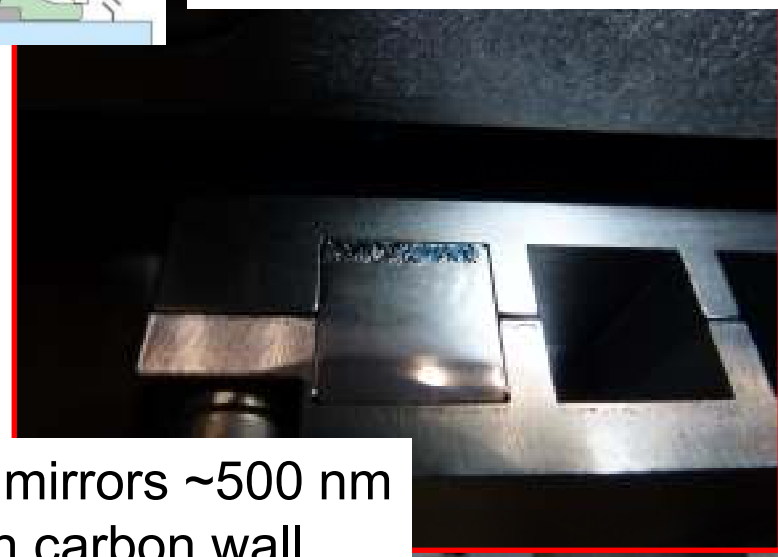
Inner Divertor

- Thin deposits – coloured fringes
- Deposits on mirror samples contain Be, W, D, Ni, C
- Be is dominant element

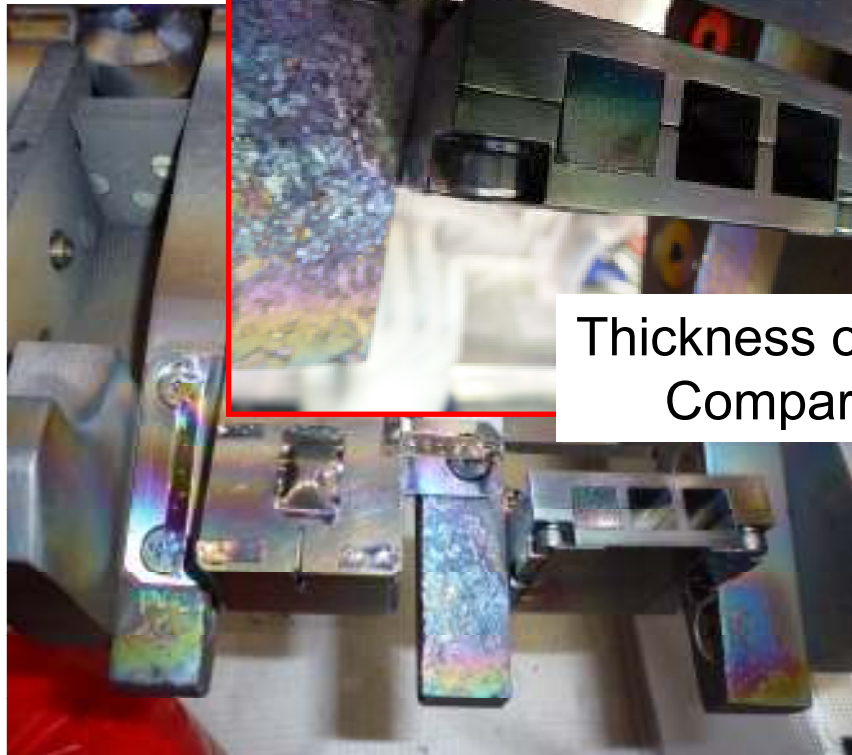


Outer Divertor

- Silvery deposits
- Deposits on mirror samples contain Be, W, D, Ni, C
- C from initial operations after intervention



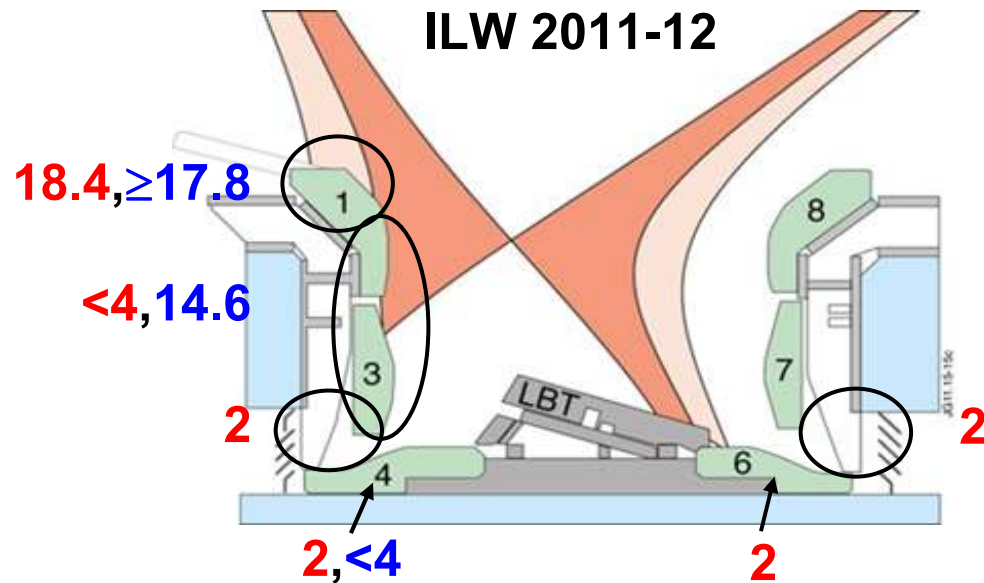
Thickness of deposits on mirrors ~500 nm
Compared to 15 μm in carbon wall



Ivanova PFMC-14
Coad PFMC-14

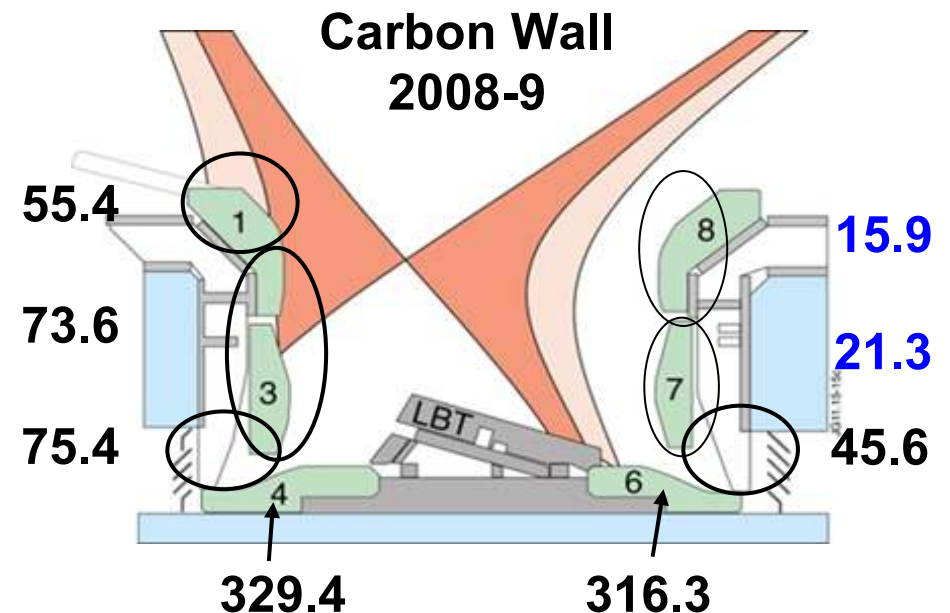


Amounts of deposit in cm^3 (extrapolated to whole divertor)



For conversion to weight (or number of atoms), may be appropriate to use density of 1 g/cm^3 for C deposits but 1.85 g/cm^3 for Be, which enhances ILW figures. Also, there may be extra Be deeper into the surface. (Coad PFMC-14)

Volume of deposits determined from
Ion Beam Analysis
Cross section microscopy
Surface profiling



Volume of deposit is 2 orders of magnitude lower in the divertor base
 Only significant deposit in ILW divertor is top of tile 1 and possible band on tile 3

- Erosion at the inner limiter during limiter plasmas
 - Mid-plane tile shows higher erosion rate per tile when compared to carbon tile
 - Fewer inner limiters in ILW
 - Rate of Be atoms eroded only slightly higher than for C
- Re-deposition during limiter plasmas in main chamber is evident but not fully quantified
 - Ends of inner and outer limiter tiles at mid-plane
 - Net deposition at top and bottom of limiters
 - Deposition on side protection tiles of outer limiters
- Deposition during X-point plasma at the divertor base tiles at least 2 orders of magnitude lower than in C wall
 - No stepwise transport via chemical erosion to migrate material to remote areas
- Exception is deposition on top of inner divertor
 - Higher deposition rate compared to carbon wall
 - Need inner wall cladding erosion data and spectroscopy to evaluate Be source during X-point plasma
 - Possible under-estimation of outer divertor erosion in carbon wall
 - Strike point location may also account for change in distribution of deposits
- Survey of tiles surfaces show the vessel wall is largely intact
 - Some areas of delamination on W-coated CFC tiles
 - Some areas of melting on Be
 - Additional erosion sources

ILW – Before operations

ILW – After operations

OPL side protection

- Be deposition on side protection tiles.
- Some delamination of W coating



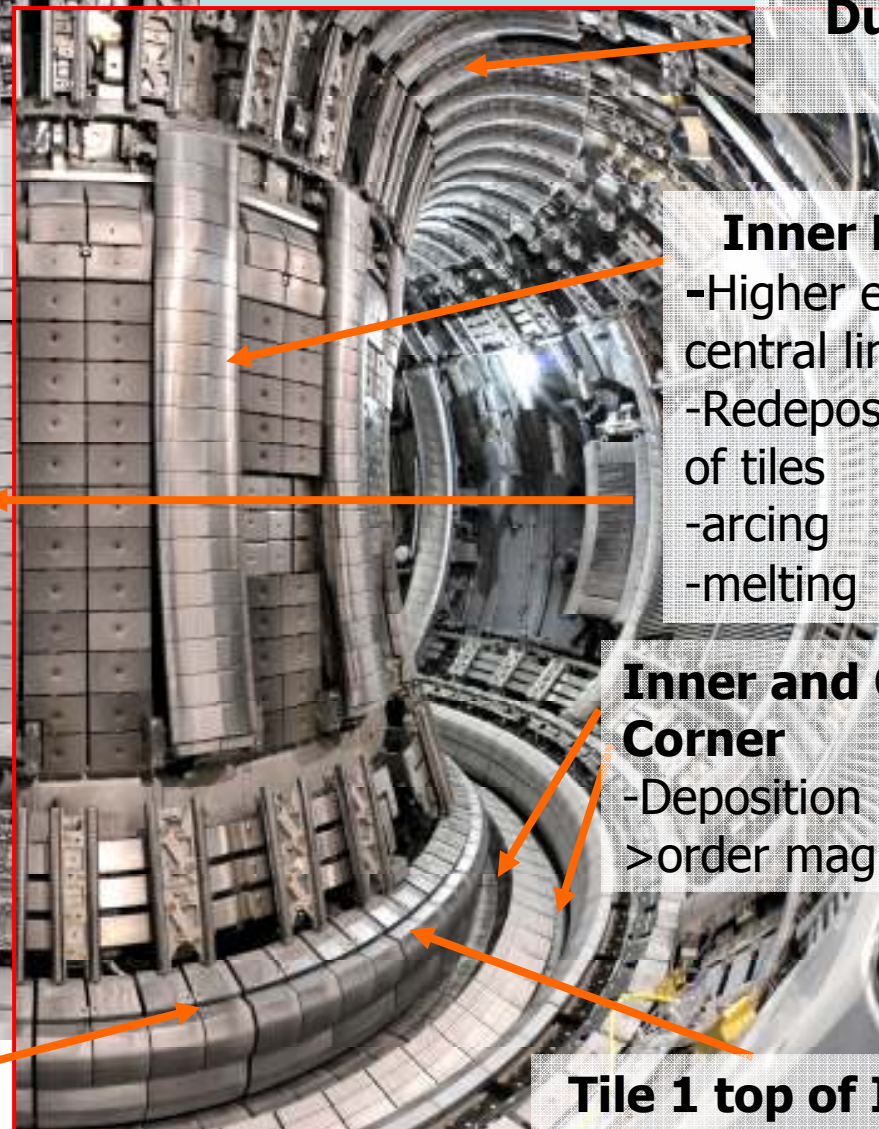
Outer Limiter Tiles

- Melting
- Evidence of erosion and re-deposition



W Coatings

- Some areas of delamination but largely intact



Dump Plates

- Melting

Inner Limiter tiles

- Higher erosion from central limiter tile
- Redeposition on ends of tiles
- arcing
- melting

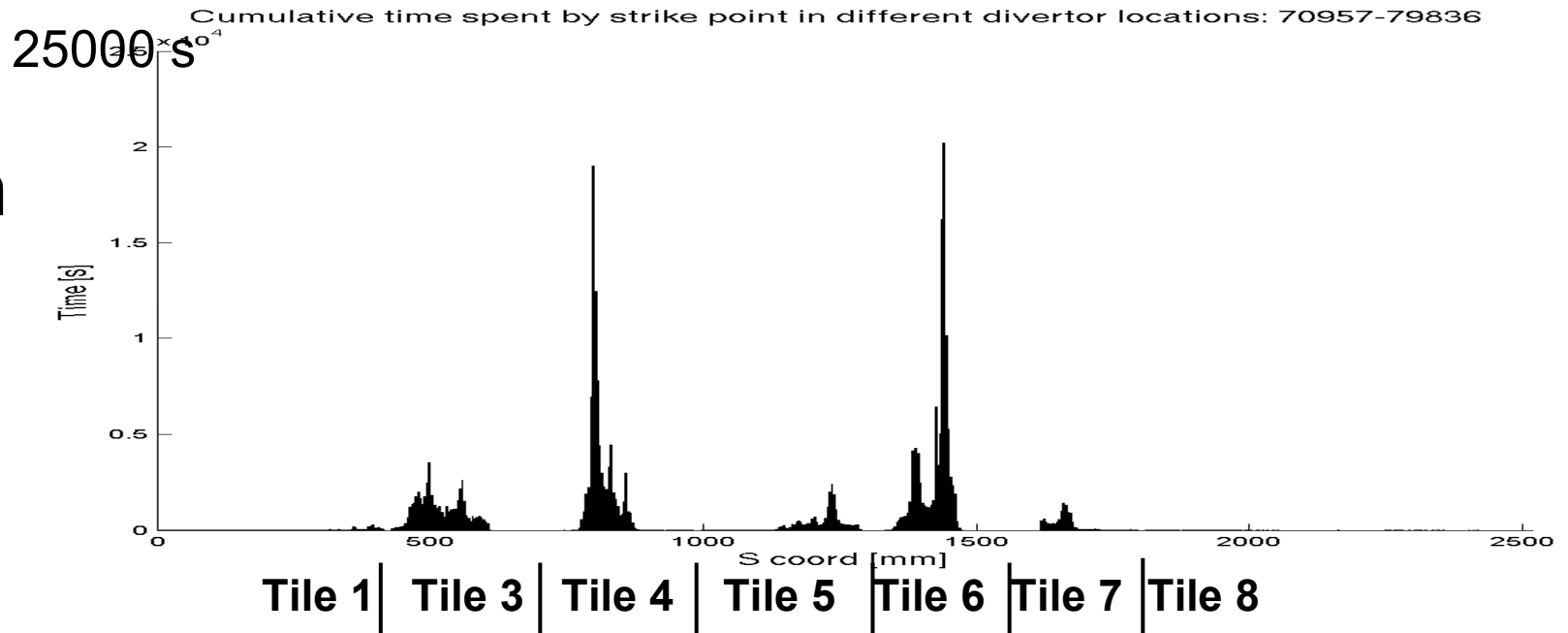
Inner and Outer Divertor Corner

- Deposition reduced by >order magnitude

Tile 1 top of Inner Divertor

- Deposition (Be) $\sim 12\mu\text{m}$

Carbon
Wall
2007-9



ILW
2010-2

